



Designation: C 753 – 99

Standard Specification for Nuclear-Grade, Sinterable Uranium Dioxide Powder¹

This standard is issued under the fixed designation C 753; the number immediately following the designation indicates the year of original adoption or, in the case of revision, the year of last revision. A number in parentheses indicates the year of last reappraisal. A superscript epsilon (ϵ) indicates an editorial change since the last revision or reappraisal.

INTRODUCTION

This specification is intended to provide the nuclear industry with a general specification for sinterable uranium dioxide powder. It recognizes the diversity of manufacturing methods by which uranium dioxide powders are produced and the many special requirements for chemical and physical characterization which may be imposed by the end use of the powder in a specific reactor system. It is, therefore, anticipated that the buyer may supplement this specification with additional requirements for specific applications.

1. Scope

1.1 This specification covers nuclear-grade, sinterable uranium dioxide powder. It applies to uranium dioxide powder containing uranium of any ^{235}U concentration for use in nuclear reactors.

1.2 This specification recognizes the presence of reprocessed uranium in the fuel cycle and consequently defines isotopic limits for commercial grade UO_2 . Such commercial grade UO_2 is defined so that, regarding fuel design and manufacture, the product is essentially equivalent to that made from unprocessed uranium. UO_2 falling outside these limits cannot necessarily be regarded as equivalent and may thus need special provisions at the fuel fabrication plant or in the fuel design.

1.3 This specification does not include provisions for preventing criticality accidents or requirements for health and safety. Observance of this specification does not relieve the user of the obligation to be aware of and conform to all international, national, or federal, state, and local regulations pertaining to possessing, shipping, processing, or using source or special nuclear material.

1.4 Special tests and procedures are given in Annex A1.

1.5 This specification refers expressly to UO_2 powder before the addition of any die lubricant, binder, or pore former. If powder is sold with such additions, sampling procedures or allowable impurity contents, or both, may need to be modified by agreement between the buyer and the seller.

1.6 The following safety hazards caveat pertains to the test methods portion in the annexes of this specification: *This*

standard does not purport to address all of the safety concerns, if any, associated with its use. It is the responsibility of the user of this standard to establish appropriate safety and health practices and determine the applicability of regulatory limitations prior to use.

2. Referenced Documents

2.1 ASTM Standards:

B 329 Test Method for Apparent Density of Powders of Refractory Metals and Compounds by the Scott Volume-ter²

C 696 Test Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis of Nuclear-Grade Uranium Dioxide Powders and Pellets³

C 859 Terminology Relating to Nuclear Materials³

C 996 Specification for Uranium Hexafluoride Enriched to Less than 5 % ^{235}U ³

C 1233 Practice for Determining Equivalent Boron Contents of Nuclear Materials³

E 11 Specification for Wire-Cloth Sieves for Testing Purposes⁴

E 105 Practice for Probability Sampling of Materials⁴

2.2 ANSI Standard:⁵

ANSI/ASME NQA-1 Quality Assurance Program Requirements for Nuclear Facilities

2.3 Federal Regulation:⁶

Code of Federal Regulations, Title 10, Chapter 1, Nuclear Regulatory Commission, Applicable Parts

² Annual Book of ASTM Standards, Vol 02.05.

³ Annual Book of ASTM Standards, Vol 12.01.

⁴ Annual Book of ASTM Standards, Vol 14.02.

⁵ Available from American National Standards Institute, 11 W. 42nd St., 13th Floor, New York, NY 10036.

⁶ Available from Standardization Documents, Order Desk, Building 4 Section D, 700 Robbins Ave., Philadelphia, PA 19111-5094, Attn: NPODS.

¹ This specification is under the jurisdiction of ASTM Committee C-26 on Nuclear Fuel Cycle and is the direct responsibility of Subcommittee C26.02 on Fuel and Fertile Material Specifications.

Current edition approved June 10, 1999. Published September 1999. Originally published as C 753 – 73. Last previous edition C 753 – 94.

3. Terminology

3.1 *Definitions*—Definitions of terms are as given in Terminology C 859.

4. Chemical Requirements

4.1 *Uranium Content*— The uranium content shall be determined on a basis to be agreed upon between the buyer and seller.

4.2 *Oxygen-to-Uranium Ratio (O/U)*—The O/U ratio may be specified as agreed upon between the buyer and seller. The determination of the O/U ratio shall be in accordance with Test Methods C 696 or demonstrated equivalent.

4.3 *Impurity Content*— The impurity content shall not exceed the individual element limit specified in Table 1 on a uranium weight basis. The summation of the contribution of each of the impurity elements listed in Table 1 shall not exceed 1500 µg/g. If an element analysis is reported as “less than” a given concentration, this “less than” value shall be used in the determination of total impurities. If an element analysis is reported as “less than” a given concentration, this “less than” value shall be used in the determination of total impurities.

4.4 *Moisture Content*— The moisture content shall not exceed 0.40 g/100 g uranium dioxide (UO₂).

4.5 *Isotopic Content:*

4.5.1 For UO₂ powder with an isotopic content of ²³⁵U between that of natural uranium and 5 %, the isotopic limits of Specification C 996 shall apply, unless otherwise agreed upon between the buyer and the seller. If the ²³⁶U content is greater than enriched commercial grade UF₆ requirements, the isotopic analysis requirements of Specification C 996 shall apply. The specific isotopic measurements required by Specification C 996 may be waived, provided that the seller can demonstrate compliance with Specification C 996, for instance, through the seller’s quality assurance records. ²³⁶U content greater than one

specified in C 996 for Commercial grade UF₆ may be agreed between the buyer and the seller since it is not a safety concern.⁷

4.5.2 For UO₂ powder, not having an assay in the range set forth in 4.5.1, the isotopic requirements shall be as agreed upon between the buyer and the seller.

4.6 *Equivalent Boron Content*—For thermal reactor use, the total equivalent boron content (EBC) shall not exceed 4.0 µg/g on a uranium weight basis. For purpose of EBC calculation B, Gd, Eu, Dy, Sm, and Cd shall be included in addition to elements listed in Table 1. The method of performing the calculation shall be as indicated in Practice C 1233. For fast reactor use, the above limitation on EBC does not apply.

5. Physical Requirements

5.1 *Particle Size*— Based on visual observation, all of a representative sample of the UO₂ shall pass through a 425-µm (No. 40) standard sieve conforming to Specification E 11. Particle size distribution and method of determination shall be as agreed upon between the buyer and seller. Alternatively, as agreed upon between the buyer and the seller, the fraction not passing through a 425-µm (No. 40) standard sieve shall be reported to the buyer.

5.2 *Bulk Density*— The bulk density of UO₂ powder will depend on the processing method. Unless otherwise agreed upon between the buyer and seller, the bulk density shall be a minimum of 625 kg/m³ as determined by Test Method B 329, or an agreed upon alternative.

5.3 *Sinterability*— Test pellets shall be produced and measured in accordance with a sintering performance test agreed upon between the buyer and seller. A sinterability performance test described in Annex A1 is presented as a guide.

6. Lot Requirements

6.1 A lot is defined as a quantity of UO₂ powder that is uniform in isotopic, chemical, physical, and sinterability characteristics.

6.2 The identity of a lot shall be retained throughout.

6.3 A powder lot shall form the basis for defining sampling plans used to establish conformance to this specification.

6.4 Sampling plans and procedures shall be mutually agreed upon by the buyer and the seller. A suggested sampling procedure is given in Annex A2.

7. Test Methods

7.1 The seller shall test the sample obtained per Annex A2 to ensure conformance of the powder to the requirements of Sections 4 and 5.

7.1.1 All chemical analyses shall be performed on portions of the representative sample prepared in accordance with Annex A2. Analytical chemistry methods used shall be in accordance with Test Methods C 696 or demonstrated equivalent methods agreed upon between the buyer and seller.

⁷ The intent of the C 996 isotope limits is to indicate possible presence of reprocessed UF₆. Acceptance of UO₂ pellets with ²³⁶U content above that specified for Commercial Enriched UF₆, shall be based on fuel performance evaluation.

TABLE 1 Impurity Elements and Maximum Concentration Limits

Element	Maximum Concentration Limit of Uranium, µg/gU
Aluminum	250
Carbon	100
Calcium + magnesium	200
Chlorine	100
Chromium	200
Cobalt	100
Copper	250
Fluorine	100
Iron	250
Lead	250
Manganese	250
Molybdenum	250
Nickel	200
Nitrogen	200
Phosphorus	250
Silicon	300
Tantalum	250
Thorium ^A	10
Tin	250
Titanium	250
Tungsten	250
Vanadium	250
Zinc	250

^AThorium is primarily of concern because of the reactor production of ²³³U.