

SLOVENSKI STANDARD SIST EN 60987:2015

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Nadomešča: SIST EN 60987:2010

Jedrske elektrarne - Merilna in nadzorna oprema za zagotavljanje varnosti - Zahteve za načrtovanje strojne opreme računalniških sistemov

Nuclear power plants - Instrumentation and control important to safety - Hardware design requirements for computer-based systems

Kernkraftwerke - Leittechnische Systeme mit sicherheitstechnischer Bedeutung -Anforderungen an die Hardware-Auslegung rechnerbasierter Systeme

Ta slovenski standard je istoveten z: EN 60987:2015

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27.120.20 Jedrske elektrarne. Varnost Nuclear power plants. Safety

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EUROPÄISCHE NORM

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English Version

Nuclear power plants - Instrumentation and control important to safety - Hardware design requirements for computer-based systems

(IEC 60987:2007 + A1:2013)

Centrales nucléaires de puissance - Instrumentation et contrôle-commande importants pour la sûreté - Exigences applicables à la conception du matériel des systèmes informatisés

(IEC 60987:2007 + A1:2013)

Kernkraftwerke - Leittechnische Systeme mit sicherheitstechnischer Bedeutung - Anforderungen an die Hardware-Auslegung rechnerbasierter Systeme (IEC 60987:2007 + A1:2013)

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European Committee for Electrotechnical Standardization Comité Européen de Normalisation Electrotechnique Europäisches Komitee für Elektrotechnische Normung

CEN-CENELEC Management Centre: Avenue Marnix 17, B-1000 Brussels

Foreword

This document (EN 60987:2015) consists of the text of IEC 60987:2007 + A1:2013 prepared by SC 45A "Instrumentation, control and electrical systems of nuclear facilities" of IEC/TC 45 "Nuclear instrumentation".

The following dates are fixed:

 latest date by which the document has to be implemented at national level by publication of an identical national standard or by endorsement
 (dop) 2016-02-16
 implemented
 at national standard or by endorsement

 latest date by which the national standards conflicting (dow) 2018-02-16 with the document have to be withdrawn

This document supersedes EN 60987:2009.

As stated in the nuclear safety directive 2009/71/EURATOM, Chapter 1, Article 2, item 2, Member States are not prevented from taking more stringent safety measures in the subject-matter covered by the Directive, in compliance with Community law. In a similar manner, this European standard does not prevent Member States from taking more stringent nuclear safety measures in the subject-matter covered by this standard.

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The text of the International Standard IEC 60987:2007 4 A1:2013 was approved by CENELEC as a European Standard without any modification. d35b/sist-en-60987-2015

In the official version, for Bibliography, the following note has to be added for the standard indicated:

IEC 61226 NOTE Harmonized as EN 61226.

Annex ZA

(normative)

Normative references to international publications with their corresponding European publications

The following documents, in whole or in part, are normatively referenced in this document and are indispensable for its application. For dated references, only the edition cited applies. For undated references, the latest edition of the referenced document (including any amendments) applies.

NOTE 1 When an International Publication has been modified by common modifications, indicated by (mod), the relevant EN/HD applies.

NOTE 2 Up-to-date information on the latest versions of the European Standards listed in this annex is available here: www.cenelec.eu

<u>Publication</u>	<u>Year</u>	<u>Title</u>	EN/HD	<u>Year</u>
IEC 60780	-	Nuclear power plants - Electrical equipment of the safety system - Qualification	-	-
IEC 60812	-	Analysis techniques for system reliability - Procedure for failure mode and effects analysis (FMEA)	EN 60812	-
IEC 60880	iT	Nuclear power plants - Instrumentation and control systems important to safety- Software aspects for computer-based systems performing category A functions	EN 60880 EW	-
IEC 61000	Series	Electromagnetic compatibility (EMC)	EN 61000	Series
IEC 61025	https://sta	Fault Tree Analysis (FTA) ndards.iteh.ai/catalog/standards/sist/cd397fbe-b7b1-4	EN 61025	-
IEC 61513	2001 ¹⁾	Nuclear power plants + Instrumentation and control for systems important to safety - General requirements for systems	-	-
IEC 62138	-	Nuclear power plants - Instrumentation and control important for safety - Software aspects for computer-based systems performing category B or C functions	EN 62138	-
IEC 62671	-	Nuclear power plants - Instrumentation and control important to safety - Selection and use of industrial digital devices of limited functionality	-	-
ISO 2768-1	-	General tolerances - Part 1: Tolerances for linear and angular dimensions without individual tolerance indications	EN 22768-1	-
ISO 2768-2	-	General tolerances - Part 2: Geometrical tolerances for features without individual tolerance indications	EN 22768-2	-

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¹⁾ Superseded by IEC 61513:2011.

<u>Publication</u>	<u>Year</u>	<u>Title</u>	EN/HD	<u>Year</u>
ISO 3951-1	-	Sampling procedures for inspection by variables - Part 1: Specification for single sampling plans indexed by acceptance quality limit (AQL) for lot-by-lot inspection for a single quality characteristic and a single AQL	-	-
ISO 3951-2	-	Sampling procedures for inspection by variables - Part 2: General specification for single sampling plans indexed by acceptance quality limit (AQL) for lot-by-lot inspection of independent quality characteristics	-	-
ISO 9001	-	Quality management systems - Requirements	EN ISO 9001	-
IAEA guide NS-G-1.3	-	Instrumentation and control systems important to safety in nuclear power plants	-	-
IAEA 50-C/SG-Q	1996	Quality assurance for safety in nuclear power plants and other nuclear installations	-	-

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IEC 60987

Edition 2.1 2013-02

INTERNATIONAL STANDARD

NORME INTERNATIONALE



Nuclear power plants—Instrumentation and control important to safety – Hardware design requirements for computer-based systems

Centrales nucléaires de puissance l'allos de la contrôle-commande importants pour la sûreté : Exigences applicables à la conception du matériel des systèmes informatisés 0467db01ad5b/sist-en-60987-2015

INTERNATIONAL
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COMMISSION

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INTERNATIONAL ELECTROTECHNICAL COMMISSION

NUCLEAR POWER PLANTS –
INSTRUMENTATION AND CONTROL
IMPORTANT TO SAFETY –
HARDWARE DESIGN REQUIREMENTS
FOR COMPUTER-BASED SYSTEMS

FOREWORD

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This consolidated version of IEC 60987 consists of the second edition (2007) [documents 45A/662/FDIS and 45A/666/RVD] and its amendment 1 (2013) [documents 45A/897/FDIS and 45A/906/RVD]. It bears the edition number 2.1.

The technical content is therefore identical to the base edition and its amendment and has been prepared for user convenience. A vertical line in the margin shows where the base publication has been modified by amendment 1. Additions and deletions are displayed in red, with deletions being struck through.

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International Standard IEC 60987 has been prepared by subcommittee 45A: Instrumentation and control of nuclear facilities, of IEC technical committee 45: Nuclear instrumentation.

This edition includes the following significant technical changes with respect to the previous edition:

- account has been taken of the fact that computer design engineering techniques have advanced significantly in the intervening years;
- update of the format to align with the current IEC/ISO directives on the style of standards;
- alignment of the standard with the new revisions of IAEA documents NS-R-1 and NS-G-1.3, which includes as far as possible an adaptation of the definitions;
- replacement, as far as possible, of the requirements associated with standards published since the first edition, especially IEC 61513, IEC 60880, edition 2, and IEC 62138;
- review of the existing requirements and updating of the terminology and definitions.

This publication has been drafted in accordance with the ISO/IEC Directives, Part 2.

The committee has decided that the contents of the base publication and its amendments will remain unchanged until the stability date indicated on the IEC web site under "http://webstore.iec.ch" in the data related to the specific publication. At this date, the publication will be

- reconfirmed,
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- withdrawn,
- replaced by a revised edition standards.iteh.ai)
- · amended.

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IMPORTANT – The "colour inside" logo on the cover page of this publication indicates that it contains colours which are considered to be useful for the correct understanding of its contents. Users should therefore print this publication using a colour printer.

INTRODUCTION

a) Technical background, main issues and organization of the standard

The basic principles for the design of nuclear instrumentation, as specifically applied to the safety systems of nuclear power plants, were first interpreted in nuclear standards with reference to hardwired systems in IAEA Safety Guide 50-SG-D3 which has been superseded by IAEA Guide NS-G-1.3.

IEC 60987 was first issued in 1989 to cover the hardware aspects of digital systems design for systems important to safety, i.e. safety systems and safety-related systems.

Although many of the requirements within the original issue continue to be relevant, there were significant factors which justified the development of this revised edition of IEC 60987, in particular:

- a new standard has been produced which addresses in detail the general requirements for nuclear systems important to safety (IEC 61513);
- the use of pre-developed system platforms, rather than bespoke developments, has increased significantly.

b) Situation of the current standard in the structure of the IEC SC 45A standard series

The first-level IEC SC 45A standard for computer-based systems important to safety in nuclear power plants (NPPs) is IEC 61513. IEC 60987 is a second-level IEC SC 45A standard which addresses the generic issue of hardware design of computerized systems.

IEC 60880 and IEC 62138 are second-level standards which together cover the software aspects of computer-based systems used to perform functions important to safety in NPPs. IEC 60880 and IEC 62138 make direct reference to IEC 60987 for hardware design.

The requirements of IEC 60780 for equipment qualification are referenced within IEC 60987. For modules to be used in the design of a specific system important to safety, relevant and auditable operating experience from nuclear or other applications as described in IEC 60780, in combination with the application of rigorous quality assurance programmes, may be an acceptable method of qualification.

For more details on the structure of the SC 45A standard series, see item d) of this introduction.

c) Recommendations and limitations regarding the application of the standard

It is important to note that this standard establishes no additional functional requirements for Class 1 or Class 2 systems (see IEC 61513 for system classification requirements).

Aspects for which special recommendations have been produced (so as to assure the production of highly reliable systems), are:

- a general approach to computing hardware development;
- a general approach to hardware verification and to the hardware aspects of computer system validation.

It is recognized that computer technology is continuing to develop and that it is not possible for a standard such as this to include references to all modern design technologies and techniques. To ensure that the standard will continue to be relevant in future years the emphasis has been placed on issues of principle, rather than specific hardware design technologies. If new design techniques are developed then it should be possible to assess the suitability of such techniques by adapting and applying the design principles contained within this standard.

The scope of this standard covers digital systems hardware for Class 1 and Class 2 systems. This includes multiprocessor distributed systems and single processor systems; it covers the assessment and use of pre-developed items, for example, commercial off-the-shelf items (COTS), and the development of new hardware.

Description of the structure of the SC 45A standard series and relationships with other IEC, IAEA and ISO documents

The top-level document of the IEC SC 45A standard series is IEC 61513. It provides general requirements for I&C systems and equipment that are used to perform functions important to safety in NPPs. IEC 61513 structures the IEC SC 45A standard series.

IEC 61513 refers direct to other IEC SC 45A standards for general topics related to categorization of functions and classification of systems, qualification, separation of systems, defence against common-cause failure, software aspects of computer-based systems, hardware aspects of computer-based systems, and control room design. The standards referenced direct at this second level should be considered together with IEC 61513 as a consistent document set.

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At a third level, IEC SC 45A standards not referenced direct by IEC 61513 are standards related to specific equipment, technical methods, or specific activities. Usually these documents, which make reference to second level documents for general topics, can be used on their own.

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A fourth level extending the IEC SC 45A standard series, corresponds to technical reports which are not normative documents.

IEC 61513 has adopted a presentation format similar to the basic safety publication IEC 61508 with an overall safety life-cycle framework and a system life-cycle framework and provides an interpretation of the general requirements of IEC 61508-1, IEC 61508-2 and IEC 61508-4, for the nuclear application sector. Compliance with IEC 61513 will facilitate consistency with the requirements of IEC 61508 as they have been interpreted for the nuclear industry. In this framework, IEC 60880 and IEC 62138 correspond to IEC 61508-3 for the nuclear application sector.

IEC 61513 refers to ISO 9001 as well as to IAEA 50-C-QA (now replaced by IAEA 50-C/SG-Q) for topics related to quality assurance (QA).

The IEC SC 45A standards series consistently implements and details the principles and basic safety aspects provided in the IAEA Code on the safety of NPPs and in the IAEA safety series, in particular the requirements of NS-R-1, establishing safety requirements related to the design of NPPs, and Safety Guide NS-G-1.3 dealing with instrumentation and control systems important to safety in NPPs. The terminology and definitions used by SC 45A standards are consistent with those used by the IAEA.