



Designation: C1268 – 94 (Reapproved 2008)

# Standard Test Method for Quantitative Determination of Americium 241 in Plutonium by Gamma-Ray Spectrometry<sup>1</sup>

This standard is issued under the fixed designation C1268; the number immediately following the designation indicates the year of original adoption or, in the case of revision, the year of last revision. A number in parentheses indicates the year of last reapproval. A superscript epsilon ( $\epsilon$ ) indicates an editorial change since the last revision or reapproval.

## 1. Scope

1.1 This test method covers the quantitative determination of americium 241 by gamma-ray spectrometry in plutonium nitrate solution samples that do not contain significant amounts of radioactive fission products or other high specific activity gamma-ray emitters.

1.2 This test method can be used to determine the americium 241 in samples of plutonium metal, oxide and other solid forms, when the solid is appropriately sampled and dissolved.

1.3 *This standard does not purport to address all of the safety concerns, if any, associated with its use. It is the responsibility of the user of this standard to establish appropriate safety and health practices and determine the applicability of regulatory limitations prior to use.*

## 2. Referenced Documents

2.1 *ASTM Standards:*<sup>2</sup>

[C758 Test Methods for Chemical, Mass Spectrometric, Spectrochemical, Nuclear, and Radiochemical Analysis of Nuclear-Grade Plutonium Metal](#)

[C759 Test Methods for Chemical, Mass Spectrometric, Spectrochemical, Nuclear, and Radiochemical Analysis of Nuclear-Grade Plutonium Nitrate Solutions](#)

[C859 Terminology Relating to Nuclear Materials](#)

[C982 Guide for Selecting Components for Energy-Dispersive X-Ray Fluorescence \(XRF\) Systems \(Withdrawn 2008\)](#)<sup>3</sup>

[C1009 Guide for Establishing a Quality Assurance Program for Analytical Chemistry Laboratories Within the Nuclear Industry](#)

<sup>1</sup> This test method is under the jurisdiction of ASTM Committee C26 on Nuclear Fuel Cycle and is the direct responsibility of Subcommittee C26.05 on Methods of Test.

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<sup>2</sup> For referenced ASTM standards, visit the ASTM website, [www.astm.org](http://www.astm.org), or contact ASTM Customer Service at [service@astm.org](mailto:service@astm.org). For *Annual Book of ASTM Standards* volume information, refer to the standard's Document Summary page on the ASTM website.

<sup>3</sup> The last approved version of this historical standard is referenced on [www.astm.org](http://www.astm.org).

[C1168 Practice for Preparation and Dissolution of Plutonium Materials for Analysis](#)

[E181 Test Methods for Detector Calibration and Analysis of Radionuclides](#)

2.2 *ANSI Standards:*<sup>4</sup>

[ANSI N15.20 Guide to Calibrating Nondestructive Assay Systems](#)

[ANSI N15.35 Guide to Preparing Calibration Material for Nondestructive Assay Systems that Count Passive Gamma Rays](#)<sup>4</sup>

[ANSI N15.37 Guide to the Automation of Nondestructive Assay Systems for Nuclear Material Control](#)<sup>4</sup>

2.3 *U.S. Nuclear Regulatory Commission Regulatory Guides:*<sup>5</sup>

[Regulatory Guide 5.9, Rev. 2—Guidelines for Germanium Spectroscopy Systems for Measurement of Special Nuclear Materials](#)

[Regulatory Guide 5.53, Rev. 1—Qualification, Calibration, and Error Estimation Methods for Nondestructive Assay](#)<sup>5</sup>

## 3. Summary of Test Method

3.1 An aliquot of the sample that contains about 10 to 100 ng of americium 241 is analyzed by measuring the intensity of the characteristic 59.5 keV gamma ray emitted by americium 241.

3.2 Multiple sample geometries may be used if an appropriate calibration for each geometry is made.

3.3 The sample geometry must be reproducible. This includes the physical characteristics of the sample container, the positioning of the sample, and the volume of sample viewed by the gamma-ray detector.

3.4 Electronic corrections are made, if required, for the effects of pulse pile-up and dead time losses due to the activity of the sample. The necessity of dead time and pulse pile-up corrections can be reduced by sample dilution to control count rates.

<sup>4</sup> Available from American National Standards Institute (ANSI), 25 W. 43rd St., 4th Floor, New York, NY 10036, <http://www.ansi.org>.

<sup>5</sup> Available from U.S. Nuclear Regulatory Commission, 1717 H ST., NW, Washington, DC 20555.

3.5 A correction is made for the contribution to the 59.5 keV intensity due to gamma rays produced in the decay of uranium 237.

3.6 The relationship between the measured gamma-ray intensity and the americium 241 content is determined by the use of appropriate standards.

#### 4. Significance and Use

4.1 This test method allows the determination of americium 241 in a plutonium solution without separation of the americium from the plutonium. It is generally applicable to any solution containing americium 241.

4.2 The americium 241 in solid plutonium materials may be determined when these materials are dissolved (see Practice C1168).

4.3 When the plutonium solution contains unacceptable levels of fission products or other materials, this method may be used following a tri-n-octylphosphine oxide (TOPO) extraction, ion exchange or other similar separation techniques (see Test Methods C758 and C759).

4.4 This test method is less subject to interferences from plutonium than alpha counting since the energy of the gamma ray used for the analysis is better resolved from other gamma rays than the alpha particle energies used for alpha counting.

4.5 The minimal sample preparation reduces the amount of sample handling and exposure to the analyst.

4.6 This test method is applicable only to homogeneous solutions. This test method is not suitable for solutions containing solids.

4.7 Solutions containing as little as  $1 \times 10^{-5}$  g/L americium 241 may be analyzed using this method. The lower limit depends on the detector used and the counting geometry. Solutions containing high concentrations may be analyzed following an appropriate dilution.

#### 5. Interferences

5.1 The presence of other radioactive nuclides in the sample or in the vicinity of the detector may produce interferences. These may be due to the Compton scattering of high energy gamma rays which contribute to the background in the region of interest or from gamma rays with energies close to the energies used for the analysis.

5.2 The presence of uranium 237 will interfere if a correction is not applied. This interference will lead to an over estimation of the amount of americium 241 present. This interference is especially pronounced in plutonium from which the americium has recently been separated.

5.3 The presence of radioactive materials in the vicinity of the gamma-ray detector which are not in the sample may create interferences if detector shielding is not adequate. These interferences may be due to the Compton scattering of high energy gamma rays which contribute to the background in the region of interest or from gamma rays with energies close to the energies used for the analysis.

#### 6. Apparatus

6.1 *High-Resolution Gamma Ray Counting System*—A high resolution gamma-ray counting system is required. General guidelines for the selection of detectors and signal processing electronics are discussed in Guide C982 and NRC Regulatory Guide 5.9. Data acquisition systems are addressed in ANSI 15.37 and NRC Regulatory Guide 5.9. This system should include the following items as a minimum.

6.1.1 *Germanium Photon Detector with Integral Preamplifier*—A coaxial type detector should typically have a full width at half maximum resolution of 850 eV or less at 122 keV and 2.0 keV or less at 1332 keV. A planar type detector should typically have a full width at half maximum resolution of 600 eV or less at 122 keV. Consideration should be given to the use of a high efficiency detector to enhance the ability to analyze low levels of americium.

6.1.2 *High Voltage Power Supply*—A high voltage power supply with voltage range and current output compatible with the detector selected is required. It is desirable that the voltage output be continuously adjustable.

6.1.3 *Nuclear Spectroscopy Amplifier*—Select a nuclear spectroscopy amplifier with pulse shaping, baseline restoration, and pulse pile-up rejection circuitry.

6.1.4 *Multichannel Pulse Height Analyzer (MCA)*—Select an MCA with a minimum of 2048 channels. It is desirable that the MCA be compatible with computerized operations so that data acquisition and analysis may be automated. The analog to digital converter (ADC) associated with the MCA should have a clock rate of at least 100 MHz and the capability of digitizing the input voltage range into a minimum of 2048 channels (other types of ADC's which provide equivalent capabilities can be used). The ADC should also have dead time and pulse pile-up correction capabilities.

6.2 *Sample Holder*, incorporating shielding to limit the interferences from background radiation sources, is required. Collimation to restrict the view of the detector to a portion of the sample may be required. The sample holder may incorporate more than one sample position. The sample holder shall provide reproducible positioning for each sample position so that a consistent volume or portion of the sample is viewed by the detector.

6.3 *Sample Vials* of sufficient volume to contain the desired sample as described in 9.2 are required. The sample vials should be made of low density materials and have reproducible dimensions such as wall thickness and internal diameter. Vials with identical dimensions should be used for samples and standards.

#### 7. Hazards

7.1 Plutonium and americium bearing materials are radioactive and toxic. Adequate laboratory facilities, gloved boxes, fume hoods, etc., along with safe techniques must be used in handling samples containing these materials. A detailed discussion of all the precautions necessary is beyond the scope of this test method; however, personnel who handle these materials should be familiar with such safe handling practices.

7.2 Solutions and solids containing radioactive materials represent a potential for high radiation exposure to personnel

handling them. Appropriate sample shielding, sample handling procedures, and radiation monitoring should be employed to ensure personnel protection.

## 8. Calibration and Standardization

8.1 Calibrate the counting system for energy (eV/channel) in the range 0 to 300 keV using a radioactive source or sources which emit gamma rays with well known energies. A plutonium source is an obvious choice. See Methods E181, ANSI N15.20, and U.S. Regulatory Guide 5.53 for further guidance.

8.2 Determine the relative detection efficiency (counts/emitted gamma ray) of the counting system in the 0 to 300 keV range. Specifically, the efficiency at 59.5 keV and 208 keV needs to be determined. See Methods E181, ANSI N15.20 and U.S. Regulatory Guide 5.53 for further guidance.

8.3 The relationship between the mass of americium 241 and the number of 59.5 keV gamma rays is established through fundamental physics and basic nuclear constants, that is, the number of 59.5 keV gamma rays/sec/gram americium 241 =  $4.543 \times 10^{10}$ .

## 9. Procedure

9.1 If necessary, prepare a plutonium solution from a solid sample following the procedure in Practice C1168 or other dissolution procedure.

9.2 Determine the amount of solution and the dilution required to provide 10 to 100 ng of americium 241 in the selected sample volume. The sample volume viewed by the detector should be consistent for the samples and standards used, regardless of the concentration.

9.3 Determine the counting time necessary to achieve the desired statistical counting precision. Samples which contain more americium will generally require less time to achieve the same statistical precision.

9.4 Quantitatively transfer the predetermined volume of solution from 9.2 into a sample vial and close.

9.5 Place the vial in the counting system sample holder and acquire a spectrum. The detector should see a consistent portion of the sample volume. The same counting geometry and sample size as used for the standards must be used.

9.6 Record the sample counting time, sample volume, dilution factor, and counting geometry used if more than one is available.

## 10. Calculation

10.1 Using the same methods as used for the calibration, determine the background corrected net count rates for the 59.5 keV gamma ray and the 208 keV gamma ray using the spectral data acquired in 9.5.

10.2 Calculate the 59.5 keV counting rate due to americium 241 in the sample.

$$R_{Am}(59) = \frac{R_{obs}(59)/D(59) - B_U R_{obs}(208)/D(208)}{1 - B_U/B_{Am}} \quad (1)$$

where:

$R_{Am}(59)$  = 59.5 keV rate (gamma rays/s) due to americium 241,

$R_{obs}(59)$  = measured 59.5 keV rate (counts/s),

$D(59)$  = detection efficiency (counts/gamma ray) at 59.5 keV,

$R_{obs}(208)$  = measured 208 keV rate (counts/s),

$D(208)$  = detection efficiency (counts/gamma ray) at 208 keV,

$B_U$  = 1.5668, and

$B_{Am}$  = 45385.6.

NOTE 1— $B_U$  and  $B_{Am}$  are dimensionless constants derived from the half-lives of uranium 237 and americium 241 and the branching ratios of the 59.5 and 208 keV gamma rays. The factor  $(1 - B_U/B_{Am})$  may be neglected for most applications.

10.3 Calculate the amount of americium 241 present in the sample using the count rate from 10.2 and the factor in 8.3.

10.4 Using the dilution factor for the sample calculate the amount of americium 241 in the original solution.

## 11. Measurement Control

11.1 Establish a measurement control program for the analytical method. Section 12 of Guide C1009 provides further guidance in this area.

11.2 As a minimum, the following periodic checks should be made.

11.2.1 Make a daily check of all instrument settings and of the energy calibration of the counting system prior to any measurement or series of measurements.

11.2.2 Make a daily measurement of the counting room background. Ideally a measurement of the room background should be made both before and after any series of americium determinations.

11.2.3 Make a daily measurement of an americium standard or sample with a known concentration to provide a measurement bias check.

11.2.4 Make weekly replicate measurements of a standard or sample to determine the precision of the measurement method.

11.3 It is recommended that control charts and other periodic statistical analysis of the precision and bias data be used.

## 12. Precision and Bias

12.1 The precision of the assay is a function of counting statistics. Precision may be improved with increased counting time.

12.2 Variations in sample vial geometry and positioning will affect the precision of the measurement.

12.3 Differences in the plutonium and acid concentration between the sample and the calibration standards may cause a bias due to self attenuation in the sample.

12.4 The calibration of standard sources, including errors introduced in using a standard radioactive solution or aliquot thereof, to prepare a working standard for bias correction may result in a bias.

12.5 The full energy peak efficiency at a given energy determined from the calibration function may introduce a bias.