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Standard Test Method for Non-Destructive Assay of Nuclear Material in Waste by Passive and Active Neutron Counting Using a Differential Die-Away System¹

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1. Scope

1.1 This test method covers a system that performs nondestructive assay (NDA) of uranium or plutonium, or both, using the active, differential die-away technique (DDT), and passive neutron coincidence counting. Results from the active and passive measurements are combined to determine the total amount of fissile and spontaneously-fissioning material in drums of scrap or waste as large as 208 L waste. Corrections are made to the measurements for the effects of neutron moderation and absorption, assuming that the effects are averaged over the volume of the drum and that no significant lumps of nuclear material are present. These systems are most widely used to assay low-level and transuranic waste, but may also be used for the measurement of scrap materials. ~~While the examples given within this test method are specific to the second-generation Los Alamos National Laboratory (LANL) passive-active neutron assay system, the principle applies to other DDT systems.~~

1.1.1 In the active mode, the system measures fissile isotopes such as ^{235}U and ^{239}Pu . The neutrons from a pulsed, 14-MeV neutron generator are thermalized to induce fission in the assay item. Between generator pulses, the system detects prompt-fission neutrons emitted from the fissile material. The number of detected neutrons between pulses is proportional to the mass of fissile material. This method is called the differential die-away technique.

1.1.2 In the passive mode, the system detects time-coincident neutrons emitted from spontaneously fissioning isotopes. The primary isotopes measured are ^{238}Pu , ^{240}Pu , and ^{242}Pu ; however, the system may be adapted for use on other spontaneously-fissioning isotopes as well, such as kilogram quantities of ^{238}U . The number of coincident neutrons detected is proportional to the mass of spontaneously-fissioning material.

1.2 The active mode is used to assay fissile material in the following ranges.

1.2.1 For uranium-only bearing items, the DDT can measure the ^{235}U content in the range from 0.02 to over 100 g. Normally, the assay of items bearing only uranium is performed using matrix-specific calibrations to account for the effect of the matrix on the active signal.

1.2.2 For plutonium-bearing items, the DDT method measures the ^{239}Pu content in the range from about 0.02 to over 100 g. Small mass uranium-bearing items are typically measured using the active mode and only large mass items are measured in passive mode.

1.2.2 For plutonium-only bearing items, the DDT method measures the ^{239}Pu content in the range between about 0.01 and 20 g.

1.3 The passive mode is capable of assaying spontaneously-fissioning nuclei, over a nominal range from 0.05 to 15 g of ^{240}Pu , or equivalent. The passive mode can also be used to measure large (for example, kg) quantities of ^{238}U -Pu equivalent.

1.4 This test method requires knowledge of the relative abundances of the plutonium or uranium isotopes to determine the total plutonium or uranium mass.

1.5 This test method will give biased results when the waste form does not meet the calibration specifications and the measurement assumptions presented in this test method regarding the requirements for a homogeneous matrix, uniform source distribution, and the absence of nuclear material lumps, to the extent that they effect the measurement.

1.6 The complete active and passive assay of a 208 L drum is nominally 10 min or less.

1.7 Improvements to this test method have been reported

1.6 The complete active and passive assay of a 208 L drum is nominally 10 min or less but either mode can be extended to meet data quality objectives.

⁺This test method is under the jurisdiction of ASTM Committee C26 on Nuclear Fuel Cycle and is the direct responsibility of Subcommittee C26.10 on Non-Destructive Assay.

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1.7 Some improvements to this test method have been reported (1, 2, 3, 4).² Discussions of these improvements are not included in this test method:

~~1.8 Discussions of these improvements are not included in this test method although improvements continue to occur.~~

1.8 The values stated in SI units are to be regarded as standard. No other units of measurement are included in this standard.

1.9 This standard may involve hazardous materials, operations, and equipment. This standard does not purport to address all of the safety concerns, if any, associated with its use. It is the responsibility of the user of this standard to establish appropriate safety and health practices and determine the applicability of regulatory limitations prior to use. Specific precautionary statements are given in Section 8.

2. Referenced Documents

2.1 ~~ASTM Standards: C859 Terminology Relating to Nuclear Materials~~

~~C986 Guide for Developing Training Programs in the Nuclear Fuel Cycle³~~

~~C1009 Guide for Establishing a Quality Assurance Program for Analytical Chemistry Laboratories within the Nuclear Industry³~~

~~C 1030 Test Method for Determination of Plutonium Isotopic Composition by Gamma-Ray Spectrometry³~~

~~C1068 Guide for Qualification of Measurement Methods by a Laboratory within the Nuclear Industry³~~

~~C1128 Guide for the Preparation of Working Reference Materials for Use in the Analysis of Nuclear Fuel Cycle Materials³~~

~~C1156 Guide for Establishing Calibration for a Measurement Method used to Analyze Nuclear fuel Cycle Materials³ Test Method for Determination of Plutonium Isotopic Composition by Gamma-Ray Spectrometry~~

~~C 1207 Test Method for Nondestructive Assay of Plutonium in Scrap and Waste by Passive Neutron Coincidence Counting³~~

~~C1210 Guide for Establishing a Measurement System Quality Control Program for Analytical Chemistry Laboratories within the Nuclear Industry³ Test Method for Nondestructive Assay of Plutonium in Scrap and Waste by Passive Neutron Coincidence Counting~~

~~C 1215 Guide for Preparing and Interpreting Precision and Bias Statements in Test Method Standards used in the Nuclear Industry³ Guide for Preparing and Interpreting Precision and Bias Statements in Test Method Standards Used in the Nuclear Industry~~

~~C 1490 Guide for the Selection, Training and Qualification of Nondestructive Assay (NDA) Personnel~~

~~C 1592 Guide for Nondestructive Assay Measurements~~

~~C 1673 Terminology of C26.10 Nondestructive Assay Methods~~

2.2 *ANSI Standard:*⁴

ANSI N15.20 Guide to Calibrating Nondestructive Assay Systems

2.3 *U.S. Government Documents:*⁵

DOE Order 435.1 (supersedes DOE Order 5820.2A) Radioactive Waste Management

DOE Order 474.1 (supersedes DOE Order 5633.3B) Control and Accountability of Nuclear Materials

DOE Order 5630.2 Control and Accountability of Nuclear Materials, Basic Principles

DOE /WIPP-069 Waste Acceptance Criteria for the Waste Isolation Pilot Plant

10 CFR Part 71 Packaging and Transport of Radioactive Materials

40 CFR Part 191 Environmental Radiation Protection Standards for Management and Disposal of Spent Nuclear Fuel, High-Level, and Transuranic Radioactive Waste

USNRC Regulatory Guide 5.11 Nondestructive Assay of Special Nuclear Materials Contained in Scrap and Waste

USNRC Regulatory Guide 5.53 Qualification, Calibration, and Error Estimation Methods for Nondestructive Assay

3. Terminology

3.1 *Definitions*—: The following definitions are needed in addition to those presented in C26.10 Terminology ~~C859~~ C 1673.

3.1.1 *active mode, n*—determines total fissile mass of the assayed item through thermal neutron interrogation and subsequent detection of prompt-fission neutrons released from induced fission. A 14-MeV neutron generator is pulsed at a nominal rate of 50 Hz. The pulsed neutrons rapidly thermalize in the chamber and in the assay item. Thermal neutrons are captured by fissile material which then fissions and immediately releases more neutrons which are detected prior to the initiation of the next pulse. The prompt-neutron count rate is proportional to the mass of fissile material. This mode is called the differential die-away technique (DDT). Refer to Fig. 1.

² The boldface numbers given in parentheses refer to a list of references at the end of the text.

³ For referenced ASTM standards, visit the ASTM website, www.astm.org, or contact ASTM Customer Service at service@astm.org. For *Annual Book of ASTM Standards* Vol 12.01 volume information, refer to the standard's Document Summary page on the ASTM website.

⁴ Available from American National Standards Institute, 11 W. 42nd St., 13th Floor, New York, NY 10036.

⁴ Available from American National Standards Institute (ANSI), 25 W. 43rd St., 4th Floor, New York, NY 10036, <http://www.ansi.org>.

⁵ Available from U.S. Government Printing Office Superintendent of Documents, 732 N. Capitol St., NW, Mail Stop: SDE, Washington, DC 20401, <http://www.access.gpo.gov>.

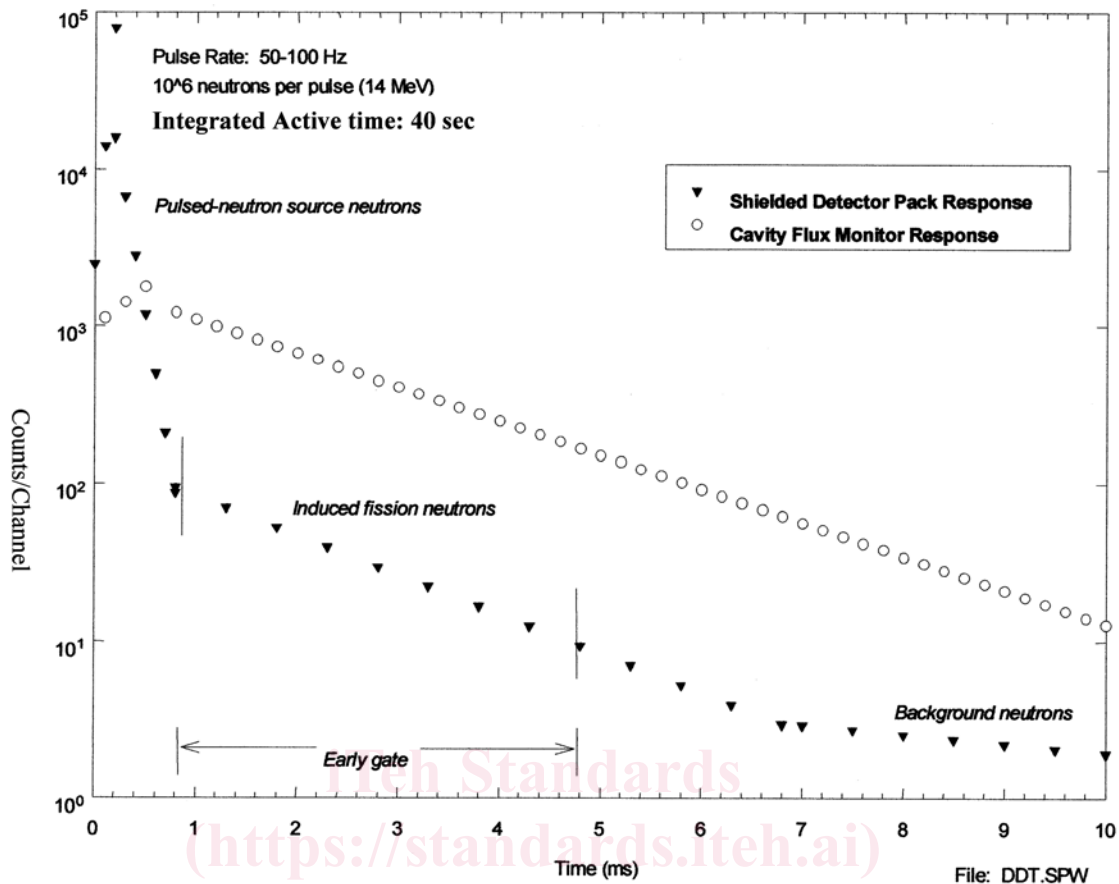


FIG. 1 Time History of an Active Assay of Plutonium Using the Differential Die-Away Technique

3.1.2 *(alpha, n) reactions*—bare detector package, n —occur when energetic alpha particles collide with low atomic number nuclei such as ^{18}O , F, or Mg producing single neutrons. Neutrons produced in this manner are not correlated in time and are a source of accidentals in passive mode and background in active mode.

3.1.3 *coincidence gate length*, n —neutron detectors surrounded by polyethylene, but not shielded with cadmium. These packages provide a better efficiency for thermal neutrons, thus providing a better passive sensitivity when a small amount of nuclear material is present.

3.1.2.1 *bare totals*, n —the time interval following the detection of a neutron during which additional neutron counts are considered to be in coincidence with the original neutron. Coincidence gate lengths are generally determined by the die-away time of the detector package. The gate length for a shielded detector package is nominally between 35 and 70 μs . The gate length for a bare detector package is nominally 250 μs .

3.1.4 *coincident neutrons*, n —two or more neutrons emitted simultaneously from a single event, such as from a nucleus during fission.

3.1.5 *combined passive and active*, n —a method which uses passive and active modes to determine the spontaneously-fissioning and fissile mass components of the waste form, respectively.

3.1.6 *depleted uranium*, n —uranium containing less than the naturally occurring fraction of ^{235}U isotopes (< 0.7 weight percent).

3.1.7 *die-away time*, n —the average lifetime of the neutron population in an NDA assay system as measured from the time of emission to detection, escape, or absorption. The average lifetime is the time required for the neutron population to decrease by a factor of $1/e$. It is a function of several parameters including chamber design, detector design, assay item characteristics, and neutron energy.

3.1.8—*is the sum of neutrons detected from all bare detector packages.*

3.1.3 *early gate*, n —the time interval during which the thermal-neutron induced prompt-fission neutrons are measured.

3.1.3.1 *Discussion*—Typically, this time interval begins 0.4 to 0.9 ms after the initiating neutron generator pulse and is 2 to 4 ms in duration. This gate is used only during the active mode. Fig. 1 indicates the approximate delay and length of the early gate in reference to a generator pulse.

3.1.9 *fissile isotopes*, n —isotopes that can be induced to fission by neutrons with thermal kinetic energy, about 0.025 electron volts. ^{233}U , ^{235}U , ^{239}Pu , and ^{241}Pu are the most common fissile isotopes.

3.1.10 *flux monitors*, n —detectors in the measurement chamber. There are two types of flux monitors:

3.1.10.1 *cavity flux monitor, n*—bare neutron detectors used to monitor the intensity of the interrogating thermal neutron flux in the chamber.

3.1.10.2 *drum flux monitors, n*—bare neutron detectors placed close to the drum and collimated with cadmium to measure the thermal neutron flux emitted from the drum.

3.1.11

3.1.4 *late gate, n*—the time interval during which the active neutron background is measured. Typically, this time interval begins 8 to 18 ms after the initiating neutron generator pulse. Refer to Fig. 1.

3.1.12 *lump, n*—that contiguous mass of nuclear material that is sufficiently large to affect the measured signal. In the active mode, self-shielding of the thermal neutron interrogating flux results in an underestimation of the fissile mass. In the passive mode, self-multiplication leads to an overestimation of the spontaneous fissioning mass.

3.1.13 *matrix, n*—the material which comprises the bulk of the item, except for the assay isotopes and the container.

3.1.14 *matrix-specific calibration, n*—uses a calibration matrix for both passive and active assays similar to the matrix to be measured. No matrix correction factors are used. This calibration is generally not appropriate for other matrices.

3.1.15 *neutron absorbers, n*—materials which have relatively large thermal neutron capture cross-sections. Absorbers with the largest capture cross-sections are commonly known as neutron poisons. Some examples are boron, cadmium, gadolinium and lithium.

3.1.16 *neutron detector package, n*—a bundle of two or three, 2- or 3-ft long neutron proportional detectors (for example, ³He tubes) surrounded by polyethylene. The output of the detectors from one package is combined into one signal and processed by a single preamplifier/amplifier/discriminator. Neutron detector packages are of two types:

3.1.16.1 *bare detector package, n*—neutron detectors surrounded by polyethylene, but not shielded with cadmium. These packages provide a better efficiency for thermal neutrons, thus providing a better passive sensitivity when a small amount of nuclear material is present.

3.1.16.2 *shielded detector package, n*—neutron detectors surrounded by polyethylene and shielded by a thin wrapping of cadmium. These packages measure the neutrons produced by nuclear interactions and are relatively insensitive to thermalized neutrons.

3.1.17 *neutron moderators, n*—materials which slow down neutrons through elastic scattering interactions. Materials containing large amounts of low atomic weight materials, for example, hydrogen, are highly moderating.

3.1.18 *passive mode, n*—a technique used to determine the spontaneously fissioning mass in the measured item through the detection of coincident neutrons. The coincident neutrons are prompt neutrons.

3.1.19 ²⁴⁰Pu-effective mass, m_{eff} , *n*—the mass of ²⁴⁰Pu that would produce the same coincident neutron response in the instrument as the assay item. It is a function of the quantity of even mass isotopes of plutonium in the assay item and fundamental nuclear constants.

3.1.20 *prompt neutrons, n*—neutrons released within approximately 10^{-14} s of the fissioning event. For example, on average, ²³⁵U and ²³⁹Pu emit 2.41 and 2.88, respectively, prompt neutrons per neutron-induced fission event. ²⁴⁰Pu emits on an average of 2.16 neutrons per spontaneous fission event.

3.1.21 *pulsed neutron generator, n*—a device which can supply a pulsed flux of neutrons. A widely used generator is the zetratron which produces 14-MeV neutrons via the deuterium-tritium interaction. Zetratrons generate a 10 to 20 μ s pulse at a frequency of 50 Hz.

3.1.22 *spontaneously fissioning nuclei, n*—those nuclei which do not require an external neutron source to undergo fission. The most common isotopes are ²³⁸Pu, ²⁴⁰Pu, ²⁴²Pu, ²⁴⁴Cm and ²⁵²Cf.

3.1.23 *totals, n*—total number of individual neutrons detected during the count time, *t*.

3.1.23.1 *bare totals, n*—is the sum of neutrons detected from all bare detector packages.

3.1.23.2 *shielded totals, n*—is the sum of neutrons detected from all shielded detector packages.

3.1.23.3 *system totals, n*—is the sum of neutrons detected in both the bare and shielded detector packages.

3.1.24 *transuranic waste, TRU waste, n*—as defined by DOE Order 435.1 (5), transuranic waste is radioactive waste containing alpha-emitting isotopes with atomic number greater than 92, half-life greater than 20 years, and with activity concentration greater than 100 nCi per gram of waste at the time of the assay.

3.1.25 *volume weighted average response, n*—an estimate of the count rate that would be obtained from a drum containing a uniform distribution of special nuclear material. It is a weighted average calculated from a series of measurements as follows: the drum is divided into typically 10 to 15 volume elements, a point source is centered in one of the volume elements and measured; the point source is moved to the next volume element and measured, and each response is weighted by the size of the corresponding element. See Appendix X1 for a more detailed explanation.

3.1.26 *wide-range calibration, n*—a calibration technique for both passive and active assays that uses calibration sources in a variety of homogeneous mock matrices whose moderation and absorption properties span the range expected to be encountered in the waste drums for which the calibration is to be used. Relationships between the measured quantities and correction factors are determined through empirical models. When the measured quantities for an “unknown” item fall within the range spanned during the calibration, the assay is within the calibration range. When an assay is not within the range spanned during the calibration, matrix-specific calibrations are performed to account for the unique properties of the item.

4. Summary of Test Method

4.1 This test method addresses a system that performs active differential die-away and passive neutron coincidence counting. Examples of the apparatus, data acquisition, and calculations contained in this test method are specific to the second-generation Los Alamos National Laboratory LANL passive-active neutron assay system (65) but the principle applies to other DDT systems.

4.1.1 Typically, the active mode is performed prior to the passive mode. A 208 L drum is placed inside the chamber and rotated continuously during the measurement. The active mode is performed by interrogating the drum with neutrons from a pulsed neutron generator for 40 to 200 s. The passive mode is performed using a counting interval of 200 to 1000 s (65, 76, 87). If the isotopic ratios as well as the relative responses are known for individual radionuclides, the active and passive modes can be used to give independent measurements of the total plutonium mass.

4.1.2 The system can also be operated only in the passive mode to measure the plutonium content of scrap or waste, or only in the active mode for measurement of uranium.

~~4.1.3 For waste containing both uranium and plutonium, determination of the mass of the fissile materials is performed outside of the standard DDT code.~~

~~4.1.4 In all modes, the relative abundances of the plutonium and uranium isotopes are required to determine the total plutonium and/or uranium mass.~~

4.1.3 In all modes, the relative abundances of the plutonium and uranium isotopes are required to determine the total plutonium mass, uranium mass, or both.

4.2 The active assay is performed using the differential die-away technique (65, 76, 87). The technique is described below and in Fig. 1.

4.2.1 A 14-MeV neutron generator is pulsed ~~repeatedly, periodically,~~ with a pulse width of 10 to 20 μ s, usually at a frequency of 50 or 100 Hz.

4.2.2 After each pulse, the neutrons are quickly moderated to thermal energies in the polyethylene ~~and/or walls,~~ graphite walls, or both, of the cavity and ultimately, in the waste matrix of the drum where they induce fission in fissile material.

4.2.3 The high energy neutrons from the generator that enter the Cd-shielded detector packages decrease in number exponentially (due to capture or escape). After about 600 to 900 μ s, essentially all of the high energy interrogating neutrons have been cleared from the detector packages and the remaining interrogating flux of neutrons is at thermal energies.

4.2.4 Fissions induced by the interrogating neutron flux in the fissile material in the drum produce prompt high-energy neutrons, which are thermalized by the waste matrix and polyethylene ~~in the walls of the sample measurement~~ chamber before being measured by the shielded detector packages during the early gate. Typically, the prompt neutrons are counted in this gate, nominally between 0.7 to 4.7 ms after each generator pulse (see Fig. 1, Region A). The temporal difference between the fission neutron signal and the tail of the interrogation neutron signal gives rise to the name of the technique - differential die-away.

4.2.5 A background count is also made during the late gate (typically 8 to 18 ms after each pulse after the moderated interrogating and induced fission neutrons have been cleared from the system (see Fig. 1, Region B). The late gate count is used to correct the early gate count for background neutrons, which are those neutrons, including delayed fission neutrons, that are not prompt fission neutrons.

4.2.6 The net number of prompt neutrons detected, normalized to the interrogating neutron flux as measured by the cavity flux monitor, is correlated to the quantity of fissile material in the drum.

4.2.7 The total nuclide mass is determined from the known relative abundances of the isotopes (Test Method C 1030) and the measured fissile mass.

4.3 The passive assay uses both shielded and bare detector packages to count accidentals and coincident neutrons from spontaneously-fissioning nuclei. Corrections are made to the counting data to account for background coincident neutrons. The number of coincident neutrons detected by the system is correlated to the mass of spontaneously-fissioning isotopes (for example, the even mass isotopes of plutonium) in the assay item (Test Method C 1207). The total plutonium mass is determined from the corrected coincidence count rates, the calibration curve correlating the corrected coincidence count rates with the ^{240}Pu -effective mass, and the known or measured plutonium isotopic ratios (Test Method C 1030).

4.4 Correction factors that account for matrix effects in the observed count rates may be calculated using the ratios of counts from the cavity flux monitor and drum flux monitor (obtained during the active measurement), and from the shielded and bare detector packages (obtained during the passive measurement).

4.4.1 Generally, both ratios can be used to correct the active and passive assay results.

4.4.2 If there is no passive result, or if the passive count rates are very low (resulting in very poor counting statistics), the correction factor obtained from the ratio of the shielded and bare detector packages is not useful. For this case, the active mode results using matrix-specific calibration factors should be used.

5. Significance and Use

5.1 This test method is useful for quantifying fissile (for example, ^{233}U , ^{235}U , ^{239}Pu and ^{241}Pu) and spontaneously-fissioning nuclei (for example, ^{238}Pu , ^{240}Pu , ^{242}Pu , ^{244}Pu , ^{248}Cm , ^{248}Cm , and ^{252}Cf) in waste and scrap drums. Total elemental mass of the radioactive materials can be calculated if the relative abundances of each radionuclide are known.

5.1.1 Typically, this test method is used to measure one fissile isotope (for example, ^{235}U or ^{239}Pu).

5.2 This test method can be used to segregate low level and transuranic waste at the $60\text{--}100$ nCi/g concentration level currently required to meet the DOE Waste Isolation Pilot Plant (WIPP) waste acceptance criterion (**65, 98, 109**).

5.3 This test method can be used for waste characterization to demonstrate compliance with the radioactivity levels specified in waste, disposal, and environmental regulations (See NRC regulatory guides, DOE Order 435.1, 10 CFR Part 71, 40 CFR Part 191, and DOE- /WIPP-069).

5.3.1 In the active mode, the DDT system can measure the ^{235}U content in the range from ≤ 0.02 to > 100 g and the ^{239}Pu content, nominally between ≤ 0.01 and ≥ 20 g.

5.3.2 In the passive mode, the DDT system is capable of assaying spontaneously-fissioning nuclei, over a nominal range from 0.05 to 15 g of ^{240}Pu , or equivalent (**65, 110, 121, 131, 141**).

5.4 This test method should be used in conjunction with a waste management plan that segregates the contents of assay items into material categories according to some or all of the following criteria: bulk density of the waste, chemical forms of the plutonium or uranium and matrix, (α, n) neutron intensity, hydrogen (moderator) and absorber content, thickness of fissile mass(es), and the assay item container size and composition. Each matrix may require a different set of calibration standards and may have different mass calibration limits. The effect on the quality of the assay (that is, minimizing precision and bias) can significantly depend on the degree of adherence to this waste management plan.

5.5 The bias of the measurement results is related to the fill height, the homogeneity and composition of the matrix, the quantity and distribution of the nuclear material, and the item size. The precision of the measurement results is related to the quantity of the nuclear material, the background, and the count time of the measurement.

5.5.1 For both matrix-specific and wide-range calibrations, this test method assumes the calibration material matches the items to be measured with respect to homogeneity and composition of the matrix, the neutron moderator and absorber content, and the quantity, distribution, and form of nuclear material, to the extent they affect the measurement.

5.5.2 The algorithms for this test method assume homogeneity. Heterogeneity in the distribution of nuclear material, neutron moderators, and neutron absorbers has the potential to cause biased results (**1514**).

5.5.3 This test method assumes that the distribution of the contributing radioisotopes is uniform throughout the container and that lumps of nuclear material are not present.

5.6 Reliable results from the application of this test method require training of the personnel who package the scrap and waste prior to measurement, and of personnel who perform the measurements. Training guidance is available from ANSI N15.20, Guides C1009, C986 and C1068. In some cases, site-specific requirements will dictate the packaging requirements.

5.5.3 This test method assumes that the distribution of the contributing radioisotopes is uniform throughout the container and that lumps of nuclear material are not present.

5.6 Reliable results from the application of this test method require waste to be packaged so the conditions of Section 5.5 can be met. In some cases, site-specific requirements will dictate the packaging requirements with possible detrimental effects to the measurement results.

5.7 Both the active mode and the passive mode provide assay values for plutonium. During the calibration process, the operator should determine the applicable mass ranges for both modes of operation.

6. Interferences

6.1 Potential sources of measurement interference include self-shielding by lumps of fissile material, unexpected nuclear material contributing to the active or passive neutron signal, non-uniform nuclear material distributions within a highly moderating matrix, heterogeneity of the matrix, excessive quantities of moderators or absorbers in the matrix, multiplication, high (α, n) rates, high count rates, cosmic rays, high neutron backgrounds, and source characteristics (for example, oxide or metal). In general, the largest potential source of bias for active neutron measurements results from heterogeneous distributions of fissile material within a highly moderating and absorbing matrix, while the largest for the passive mode measurements is excessive neutron absorption.

6.1.1 The techniques used in this test method cannot distinguish which isotope is generating the measured response. If more than one neutron-producing nuclide is present, the relative abundances and relative responses of those radionuclides must be known:

6.1.1.1

6.1 Potential sources of measurement interference include:

6.1.1 self-shielding by lumps of fissile material,

6.1.2 unexpected nuclear material contributing to the active or passive neutron signal,

6.1.3 non-uniform nuclear material distributions within a moderating matrix,

6.1.4 heterogeneity of the matrix,

6.1.5 excessive quantities of moderators or absorbers in the matrix,

6.1.6 multiplication, high (α, n) rates,

6.1.7 high count rates, cosmic rays, and

6.1.8 high neutron backgrounds.

6.2 The techniques used in this test method cannot distinguish which isotope is generating the measured response. If more than one neutron-producing nuclide is present, the relative abundances and relative responses of those radionuclides must be known.

6.2.1 *Active Mode*—The presence of other fissile radionuclides will increase the induced fission neutron count rate, causing an over-estimation of the ^{235}U or ^{239}Pu content, unless a correction is made. Induced fission neutrons from ^{235}U , ^{239}Pu and ^{241}Pu are