

SLOVENSKI STANDARD SIST EN ISO 18417:2019

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Jodni ogleni sorbenti za jedrske objekte - Metoda za določanje indeksa sorpcijske zmogljivosti (ISO 18417:2017)

lodine charcoal sorbents for nuclear facilities - Method for defining sorption capacity index (ISO 18417:2017)

Jodkohlenstoffsorptionsmittel für kerntechnische Anlagen - Verfahren zur Bestimmung des Sorptionsvermögensindexes (ISO 18417:2017) PREVIEW

Pièges à iode pour installations nucléaires - Méthode pour définir la capacité de rétention (ISO 18417:2017)

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Iodine charcoal sorbents for nuclear facilities - Method for defining sorption capacity index (ISO 18417:2017)

Pièges à iode pour installations nucléaires - Méthode pour définir la capacité de rétention (ISO 18417:2017)

Jodkohlenstoffsorptionsmittel für kerntechnische Anlagen - Verfahren zur Bestimmung des Sorptionsvermögensindexes (ISO 18417:2017)

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EN ISO 18417:2019 (E)

European foreword

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Iodine charcoal sorbents for nuclear facilities — Method for defining sorption capacity index

Pièges à iode pour installations nucléaires — Méthode pour définir la capacité de rétention

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Foreword

ISO (the International Organization for Standardization) is a worldwide federation of national standards bodies (ISO member bodies). The work of preparing International Standards is normally carried out through ISO technical committees. Each member body interested in a subject for which a technical committee has been established has the right to be represented on that committee. International organizations, governmental and non-governmental, in liaison with ISO, also take part in the work. ISO collaborates closely with the International Electrotechnical Commission (IEC) on all matters of electrotechnical standardization.

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Introduction

lodine sorbents are extensively used in nuclear facilities to remove radioiodine from gases and air in off-gas cleaning systems and ventilation installations. The sorbents are very important for protection of the members of the public and environment from iodine radionuclides radiation.

In the normal operation of nuclear installations, the main hazard comes from radioactive isotopes of iodine; as examples, for reactors 131 I and a minor extent 133 I, for fuel processing facilities 129 I, etc. Iodine is one of the main contributors of the radiation impact on the environment. Under abnormal and accident conditions, some other isotopes 132 I, 134 I and 135 I have also some significant effects on the total iodine dose (thyroid dose)[3].

The volatile radioiodine forms can occur in the gaseous radioactive wastes as elemental iodine, the simplest organic compound methyl iodide, and some others such as hydrogen iodide under reducing conditions.

Radioactive iodine can create a serious danger to the members of the public and workers in abnormal and accident conditions at nuclear facilities as far as the exposure in these conditions could be much higher than the exposure due to the natural background radiation.

The need to prevent widespread dispersal of gaseous radioiodine from nuclear facilities is a major purpose of iodine sorbents. It is universally recognized that radioactive methyl iodide is the less readily removable radioiodine form. The removal of radioactive iodine from gaseous radioactive wastes at nuclear facilities is almost always performed with the help of impregnated activated charcoals that have become often accepted as the preferred lodine sorbents used in these facilities. Impregnated charcoals require a high efficiency especially from humid gases containing iodine in order to trap all the iodine gaseous compounds. (standards.iteh.ai)

Two types of tests are considered [2][4]: laboratory and in situ tests.

- Laboratory tests are done to establish the performance characteristics of the charcoal to be used in retention systems under specified operating conditions 119
- *In situ* tests are done to obtain a measure of the performance of retention systems under appropriate operational conditions.

This document concerns only the laboratory tests. Laboratory tests of representative samples of charcoal (e.g. new charcoal, aged charcoal from iodine absorbers, etc.) are performed to establish their efficiency for a given test agent under specified conditions.

The quality of sorbents and its potential application at nuclear facilities can be estimated by means of a criterion that defines specifically the sorption capacity of the sorbent. Such criterion is called in this document the sorption capacity index.

The index is defined by the result of a laboratory test on the basis of radioiodine activity distribution inside the sorbent. This index characterizes the total kinetic sorption process for established test conditions and show whether the sorbent can be used as iodine filters for nuclear facilities. One example of criteria is given in Annex D.

This document provides a method to determine the quantitative quality of a sorbent and also to compare the performance of different iodine sorbents at the specified conditions. It is useful for users of iodine sorbents (filter or sorbent manufacturers as well as operators).

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