

SLOVENSKI STANDARD oSIST prEN ISO 16647:2021

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Jedrski objekti - Merila za načrtovanje in delovanje zadrževalnih sistemov za jedrska delovišča in jedrske naprave, ki so v razgradnji (ISO 16647:2018)

Nuclear facilities - Criteria for design and operation of confinement systems for nuclear worksite and for nuclear installations under decommissioning (ISO 16647:2018)

Kerntechnische Anlagen - Kriterien für die Planung und den Betrieb von Rückhaltesystemen und Lüftungssysteme für kurzzeitige Arbeitsplätze und für kerntechnische Anlagen, die rückgebaut werden (ISO 16647:2018)

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Installations nucléaires - Critères pour la conception et l'exploitation des systèmes de confinement des chantiers nucléaires et des installations nucléaires en démantèlement (ISO 16647:2018) (ISO 16647:2018)

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INTERNATIONAL STANDARD

ISO 16647

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Nuclear facilities — Criteria for design and operation of confinement systems for nuclear worksite and for nuclear installations under decommissioning

Installations nucléaires — Critères pour la conception et l'exploitation des systèmes de confinement des chantiers nucléaires et **iTeh ST**des installations nucléaires en démantèlement

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Foreword

ISO (the International Organization for Standardization) is a worldwide federation of national standards bodies (ISO member bodies). The work of preparing International Standards is normally carried out through ISO technical committees. Each member body interested in a subject for which a technical committee has been established has the right to be represented on that committee. International organizations, governmental and non-governmental, in liaison with ISO, also take part in the work. ISO collaborates closely with the International Electrotechnical Commission (IEC) on all matters of electrotechnical standardization.

The procedures used to develop this document and those intended for its further maintenance are described in the ISO/IEC Directives, Part 1. In particular the different approval criteria needed for the different types of ISO documents should be noted. This document was drafted in accordance with the editorial rules of the ISO/IEC Directives, Part 2 (see www.iso.org/directives).

Attention is drawn to the possibility that some of the elements of this document may be the subject of patent rights. ISO shall not be held responsible for identifying any or all such patent rights. Details of any patent rights identified during the development of the document will be in the Introduction and/or on the ISO list of patent declarations received (see www.iso.org/patents).

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For an explanation on the voluntary nature of standards, the meaning of ISO specific terms and expressions related to conformity assessment, as well as information about ISO's adherence to the World Trade Organization (WTO) principles in the Technical Barriers to Trade (TBT) see the following URL: www.iso.org/iso/foreword.html.ncards.iten.ai)

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Nuclear facilities — Criteria for design and operation of confinement systems for nuclear worksite and for nuclear installations under decommissioning

1 Scope

This document specifies the requirements applicable to the design and use of airborne confinement systems that ensure safety and radioprotection functions in nuclear worksites and in nuclear installations under decommissioning to protect from radioactive contamination produced: aerosol or gas.

The purpose of confinement systems is to protect the workers, members of the public and environment against the spread of radioactive contamination resulting from operations in nuclear worksites and from nuclear installations under decommissioning.

The confinement of nuclear worksites and of nuclear installations under decommissioning is characterized by the temporary and evolving (dynamic) nature of the operations to be performed. These operations often take place in area not specifically designed for this purpose.

This document applies to maintenance or upgrades at worksites which fit the above definition.

The requirements for the design and use of ventilation and confinement systems and for liquid NOTE confinement in nuclear reactors or in nuclear installations other than nuclear worksites and nuclear installations under decommissioning are developed in other ISO standards 1.21)

oSIST prEN ISO 16647:2021 Normative references https://standards.iteh.ai/catalog/standards/sist/65e1d5d8-86a2-4fad-9613-2

The following documents are referred to in the text in such a way that some or all of their content constitutes requirements of this document. For dated references, only the edition cited applies. For undated references, the latest edition of the referenced document (including any amendments) applies.

ISO 16170, In situ test methods for high efficiency filter systems in industrial facilities

3 **Terms and definitions**

For the purposes of this document, the following terms and definitions apply.

ISO and IEC maintain terminological databases for use in standardization at the following addresses:

ISO Online browsing platform: available at https://www.iso.org/obp

IEC Electropedia: available at http://www.electropedia.org/

3.1

climatic shelter

shelter whose function is to provide suitable protection against the weather (sun, rain, wind, snow and extreme temperatures), usually structurally separated from radiological containment

3.2

aerosol

solid particles and liquid droplets of all dimensions in suspension in a gaseous fluid

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3.3

barrier

structural element, which defines the physical limits of a volume with a particular radiological environment and which prevents or limits releases of radioactive substances from this volume

EXAMPLE Containment enclosure, shielded cell, filters.

3.4

discharge stack

duct (usually vertical) at the termination of a system, from which the air is discharged to the atmosphere

3.5

air conditioning

arrangement allowing the sustainment of a controlled atmosphere (temperature, humidity, pressure, dust levels, gas content, etc.) in a closed volume

3.6

confinement

arrangement allowing users to maintain separate environments inside and outside an enclosure, blocking the movement between them, of process materials and substances resulting from physical and chemical reactions which are potentially harmful to workers, the external environment, or to the handled products

Note 1 to entry: The word "confinement" is used in several IAEA documents to mean the function of confining radioactive or toxic products whereas "containment" is used to mean the physical barrier that achieves the objective of confinement, i.e. a confined area. TANDARD PREVIEW

3.7

worksite containment

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specific containment implemented to cover the temporary and evolving nature of worksite activities <u>oSIST prEN ISO 16647:2021</u>

3.8

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dynamic confinement

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action allowing, by maintaining a preferential air flow circulation, to limit back-flow between two areas or between the inside and outside of an enclosure, in order to prevent radioactive substances being released from a given physical volume

3.9

contamination

presence of radioactive substances on or in a material or a human body or any place where they are undesirable or could be harmful

3.10

containment enclosure

enclosure designed to prevent either the leakage of products contained in the pertinent internal environment into the external environment, or the penetration of substances from the external environment into the internal environment, or both simultaneously

3.11

gas cleaning

action of decreasing the content of undesirable constituents in a fluid

Note 1 to entry: Gas cleaning is sometimes called "scrubbing".

Note 2 to entry: Aerosol filtration and iodine trapping are examples of gas cleaning.

3.12

filter

device intended to trap particles suspended in gases and fluids or to trap gases themselves

3.13 high efficiency particle air filter HEPA filter

aeorosol filter that corresponds to the classes H35, H40 or H45 according to ISO 29463-1

3.14

last filtration stage

LFS

last filtering stage implemented on the dynamic confinement release network protecting the environment

EXAMPLE HEPA filters for aerosols, iodine filters, etc.

3.15

Derived air concentration

DAC

amount of contamination in air, which, if 2 200 ${\rm m}^3$ is inhaled, would result in the annual limit of intake (ALI)

Note 1 to entry: DAC is defined in ICRP 103 and expressed in Bq/m³.

Note 2 to entry: The ALI is calculated using reference conversion factors given by ICRP (International Commission for Radiological Protection) for each radionuclide (ICRP 119).

3.16

airtight bag ventilated airtight bag

flexible containment used to establish an enclosure around a contaminated item, allowing personnel to accomplish works or manipulations potentially via gloved sleeves without contacting the contaminated environment

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Note 1 to entry: The airtight bag may include inlet and extract ventilation in order to achieve an air velocity in leakage points or negative pressure within the containment 6647-2021

3.17

spark arrestor

device fitted upstream of the main filters to minimize transport of particles and the deterioration of main filters, by capture of incandescent large particles

3.18

prefilter

filter fitted upstream of the main air filters to minimize the dust burden on the latter, by removal of large particles

3.19

negative pressure

depression

pressure difference between the pressure of a given volume, which is maintained lower than the pressure in a reference volume or the external ambient pressure

3.20

confinement system

system constituted by a coherent set of physical barriers and/or dynamic systems intended to confine radioactive substances

3.21

ventilation system

totality of network components such as ducts, fans, filter units and other equipment, that ensures ventilation and gas cleaning functions

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3.22

air-change rate

ratio between the ventilation air flow rate of a containment enclosure or a compartment and the volume of this containment enclosure or compartment, during normal operating conditions

3.23

ventilation

organization of air flow patterns within an installation

4 Functions ensured by the confinement

The confinement of nuclear worksite and nuclear installations under decommissioning (sometimes in complement with the existing confinement of the installation) enables the improved safety of the workers, members of the public and provides protection of the environment. It plays the role of:

- Safety and radioprotection, by contributing to limit the contamination impact on the workers, members of the public and the environment.
- **Protection of equipment and rooms**, maintaining the level of cleanliness to avoid any radiological releases of contamination.

Confinement system ensures the following main functions:

- Confinement, by acting in a static and/or dynamic manner. The role of this function is to control
 the release and spread of radioactive products, in aerosol or gas form, in environment, and to
 protect workers, in particular those that do not have respiratory protection from existing volume
 radioactivity or volume radioactivity generated by activities.
- Cleaning the atmosphere of the enclosure or room, by renewing the volumes of air within it, in order to minimise the risks associated with the corresponding atmosphere (for example, the elimination of any gas that can lead to an explosion hazard, fume gas evacuation, etc.)-9613-
- Purification (or gas cleaning) by conveying the collected gases including any dust, aerosols and volatile components, to defined and controlled points for collection, processing and elimination if possible (by using filters, traps, etc.).
- Radiological cleanliness maintaining the level of the atmospheric and surface contamination of equipment and rooms, as low as possible

The dynamic confinement system may also contribute to the following functions:

- Surveillance of the releases, in particular when the static containment faces the environment by
 orientating the airflows to the contamination sensors to the exhaust points.
- **Conditioning** of the atmosphere of considered volumes to ensure ambient conditions continually compatible with the proper functioning of the equipment.

5 Principles for radioactive substances confinement

5.1 General principles

Confinement systems shall ensure the safety and radioprotection functions defined in <u>Clause 4</u>, in all normal operating conditions of nuclear worksites and nuclear installations under decommissioning. They shall also ensure that these functions continue during abnormal operating conditions, or accident situations that are to be defined case by case depending on the safety analysis.

Before beginning any confinement design, a risk assessment shall be made so that actual targets are adequately defined. <u>5.2</u> provides an outline of the risk assessment process.

5.2 Risk assessment procedure

The design of an appropriate confinement system requires preliminary analysis, taking into account:

- radiological hazards generated by materials and operations leading to the need to confine the rooms
 or work areas where hazardous substances are handled, including:
 - permissible levels of surface or airborne contamination inside the room or rooms where are contained confined enclosures;
 - requirements for airborne contamination monitoring;
- verification of discharge authorization limits in respect of actual discharges through existing ventilation systems or ventilation systems to be set up;
- risks associated with the facility to which the confined enclosures and ventilation systems can be exposed and that can be considered plausible on the installation (e.g. load drop, fire, flood, external explosion, earthquakes, wind and extreme temperatures, etc.);
- human activities deployed nearby facilities (collocated operations);
- possible temporary unavailability of fluids or energy necessary for the proper functioning of the confinement system (electricity, compressed air, neutral gases, cooling water, etc.);
- non-radiological hazards associated with equipment and operations implemented in confined enclosures (e.g. sudden break of containment due to mechanical failure, sudden change in pressure, over pressure risks, explosion, fire, corrosion, condensation, load drop), which consequences may be resuspension of radioactivity. As an example, when the worksite confinement is used in the fire safety demonstration, special analyses are needed for cases where fire extinguishers are likely to create a breach in the confinement, e.g. by pressurizing the static confinement because of their potential impact on dynamic confinement or for glove boxes for which water cannot be used when they are criticality risks ands. iteh ai/catalog/standards/sist/65e1d5d8-86a2-4fad-9613-

For each consideration, a risk assessment is to be carried out using the safety analysis methodology where the risk is defined as the combination of the consequences of the event and its estimated frequency. This may consist of a deterministic approach, based on incidental or accidental conservative situations.

Other factors to consider in the design of confinement systems are:

- to reduce the amount of waste produced and radioactive release (liquid and gaseous) to a level as low as reasonably achievable, for the protection of the environment;
- to minimize the level of contamination in the rooms or work areas as far as reasonably achievable, in particular by implementing dynamic confinement as close as possible to the source;
- the impact on the existing installation of modifications of ventilation network, enclosure, containment enclosure layout, etc.;
- physical and radiological state of the existing installation (e.g. for static confinement, cable, drains);
- incidental or accidental situations;
- appropriate work conditions that should be provided to workers;
- robustness of confinement system (e.g. fan redundancy), if considering worksite containment with high permanent volumic activity.

The risk assessment procedure is needed to define the requirements for the worksite confinement provisions and to give appropriate health physics coverage to the workers prior to the start of the activities: e.g. process provisions/rinsing/cleaning of systems to be removed or decommissioned, additional local shielding, access control.