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## Standard Guide for Determining Neutron Energy Spectra from Neutron Sensors for Radiation-Hardness Testing of Electronics<sup>1</sup>

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*This standard has been approved for use by agencies of the U.S. Department of Defense.*

### 1. Scope

1.1 This guide covers procedures for determining the energy-differential fluence spectra of neutrons used in radiation-hardness testing of electronic semiconductor devices. The types of neutron sources specifically covered by this guide are fission or degraded energy fission sources used in either a steady-state or pulse mode.

1.2 This guide provides guidance and criteria that can be applied during the process of choosing the spectrum adjustment methodology that is best suited to the available data and relevant for the environment being investigated.

1.3 This guide is to be used in conjunction with Guide E720 to characterize neutron spectra and is used in conjunction with Practice E722 to characterize damage-related parameters normally associated with radiation-hardness testing of electronic-semiconductor devices.

NOTE 1—Although Guide E720 only discusses activation foil sensors, any energy-dependent neutron-responding sensor for which a response function is known may be used (1).<sup>2</sup>

NOTE 2—For terminology used in this guide, see Terminology E170.

1.4 The values stated in SI units are to be regarded as standard. No other units of measurement are included in this standard.

1.5 *This standard does not purport to address all of the safety concerns, if any, associated with its use. It is the responsibility of the user of this standard to establish appropriate safety and health practices and determine the applicability of regulatory limitations prior to use.*

<sup>1</sup> This guide is under the jurisdiction of ASTM Committee E10 on Nuclear Technology and Applications and is the direct responsibility of Subcommittee E10.07 on Radiation Dosimetry for Radiation Effects on Materials and Devices.

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<sup>2</sup> The boldface numbers in parentheses refer to the list of references at the end of this guide.

### 2. Referenced Documents

#### 2.1 ASTM Standards:<sup>3</sup>

- E170 Terminology Relating to Radiation Measurements and Dosimetry
- E261 Practice for Determining Neutron Fluence, Fluence Rate, and Spectra by Radioactivation Techniques
- E262 Test Method for Determining Thermal Neutron Reaction Rates and Thermal Neutron Fluence Rates by Radioactivation Techniques
- E263 Test Method for Measuring Fast-Neutron Reaction Rates by Radioactivation of Iron
- E264 Test Method for Measuring Fast-Neutron Reaction Rates by Radioactivation of Nickel
- E265 Test Method for Measuring Reaction Rates and Fast-Neutron Fluences by Radioactivation of Sulfur-32
- E266 Test Method for Measuring Fast-Neutron Reaction Rates by Radioactivation of Aluminum
- E393 Test Method for Measuring Reaction Rates by Analysis of Barium-140 From Fission Dosimeters
- E523 Test Method for Measuring Fast-Neutron Reaction Rates by Radioactivation of Copper
- E526 Test Method for Measuring Fast-Neutron Reaction Rates by Radioactivation of Titanium
- E704 Test Method for Measuring Reaction Rates by Radioactivation of Uranium-238
- E705 Test Method for Measuring Reaction Rates by Radioactivation of Neptunium-237
- E720 Guide for Selection and Use of Neutron Sensors for Determining Neutron Spectra Employed in Radiation-Hardness Testing of Electronics
- E722 Practice for Characterizing Neutron Fluence Spectra in Terms of an Equivalent Monoenergetic Neutron Fluence for Radiation-Hardness Testing of Electronics

<sup>3</sup> For referenced ASTM standards, visit the ASTM website, [www.astm.org](http://www.astm.org), or contact ASTM Customer Service at [service@astm.org](mailto:service@astm.org). For *Annual Book of ASTM Standards* volume information, refer to the standard's Document Summary page on the ASTM website.

- E844 Guide for Sensor Set Design and Irradiation for Reactor Surveillance, E 706 (IIC)
- E944 Guide for Application of Neutron Spectrum Adjustment Methods in Reactor Surveillance, E 706 (IIA)
- E1018 Guide for Application of ASTM Evaluated Cross Section Data File, Matrix E706 (IIB)
- E1297 Test Method for Measuring Fast-Neutron Reaction Rates by Radioactivation of Niobium
- E1855 Test Method for Use of <sup>2N2222A</sup> Silicon Bipolar Transistors as Neutron Spectrum Sensors and Displacement Damage Monitors

### 3. Terminology

3.1 *Definitions*: The following list defines some of the special terms used in this guide:

3.1.1 *effect*—the characteristic which changes in the sensor when it is subjected to the neutron irradiation. The effect may be the reactions in an activation foil.

3.1.2 *response*—the magnitude of the effect. It can be the measured value or that calculated by integrating the response function over the neutron fluence spectrum. The response is an integral parameter. Mathematically, the response,  $R = \sum_i R_i$ , where  $R_i$  is the response in each differential energy region at  $E_i$  of width  $\Delta E_i$ .

3.1.3 *response function*—the set of values of  $R_i$  in each differential energy region divided by the neutron fluence in that differential energy region, that is, the set  $f_i = R_i/(\Phi(E_i)\Delta E_i)$ .

3.1.4 *sensor*—an object or material (sensitive to neutrons) the response of which is used to help define the neutron environment. A sensor may be an activation foil.

3.1.5 *spectrum adjustment*—the process of changing the shape and magnitude of the neutron energy spectrum so that quantities integrated over the spectrum agree more closely with their measured values. Other physical constraints on the spectrum may be applied.

3.1.6 *trial function*—a neutron spectrum which, when integrated over sensor response functions, yields calculated responses that can be compared to the corresponding measured responses.

3.1.7 *prior spectrum*—an estimate of the neutron spectrum obtained by transport calculation or otherwise and used as input to a least-squares adjustment.

3.2 *Abbreviations*:

3.2.1 *DUT*—device under test.

3.2.2 *ENDF*—evaluated nuclear data file.

3.2.3 *NNDC*—National Nuclear Data Center (at Brookhaven National Laboratory).

3.2.4 *RSICC*—Radiation Safety Information Computation Center (at Oak Ridge National Laboratory).

3.2.5 *TREE*—transient radiation effects on electronics.

### 4. Significance and Use

4.1 It is important to know the energy spectrum of the particular neutron source employed in radiation-hardness testing of electronic devices in order to relate radiation effects with device performance degradation.

4.2 This guide describes the factors which must be considered when the spectrum adjustment methodology is chosen and implemented. Although the selection of sensors (foils) and the determination of responses (activities) is discussed in Guide E720, the experiment should not be divorced from the analysis. In fact, it is advantageous for the analyst conducting the spectrum determination to be closely involved with the design of the experiment to ensure that the data obtained will provide the most accurate spectrum possible. These data include the following: (1) measured responses such as the activities of foils exposed in the environment and their uncertainties, (2) response functions such as reaction cross sections along with appropriate correlations and uncertainties, (3) the geometry and materials in the test environment, and (4) a trial function or prior spectrum and its uncertainties obtained from a transport calculation or from previous experience.

### 5. Spectrum Determination With Neutron Sensors

5.1 *Experiment Design*:

5.1.1 The primary objective of the spectrum characterization experiment should be the acquisition of a set of response values (activities) from effects (reactions) with well-characterized response functions (cross sections) with responses which adequately define (as a set) the fluence values at energies to which the device to be tested is sensitive. For silicon devices in fission-driven environments the significant neutron energy range is usually from 10 keV to 15 MeV. Lists of suitable reactions along with approximate sensitivity ranges are included in Guide E720. Sensor set design is also discussed in Guide E844. The foil set may include the use of responses with sensitivities outside the energy ranges needed for the DUT to aid in interpolation to other regions of the spectrum. For example, knowledge of the spectrum below 10 keV helps in the determination of the spectrum above that energy.

5.1.2 An example of the difficulty encountered in ensuring response coverage (over the energy range of interest) is the following: If fission foils cannot be used in an experiment because of licensing problems, cost, or radiological handling difficulties (especially with <sup>235</sup>U, <sup>237</sup>Np or <sup>239</sup>Pu), a large gap may be left in the foil set response between 100 keV and 2 MeV—a region important for silicon and gallium arsenide damage (see Figs. A1.1 and A2.3 of Practice E722). In this case two options are available. First, seek other sensors to fill the gap (such as silicon devices sensitive to displacement effects (see Test Method E1855)), <sup>93</sup>Nb(*n,n'*) <sup>93m</sup>Nb (see Test Method E1297) or <sup>103</sup>Rh(*n,n'*) <sup>103m</sup>Rh. Second, devote the necessary resources to determine a trial function that is close to the real spectrum. In the latter case it may be necessary to carry out transport calculations to generate a prior spectrum which incorporates the use of uncertainty and covariance information.

5.1.3 Other considerations that affect the process of planning an experiment are the following:

5.1.3.1 Are the fluence levels low and of long duration so that only long half-life reactions are useful? This circumstance can severely reduce the response coverage of the foil set.

5.1.3.2 Are high gamma-ray backgrounds present which can affect the sensors (or affect the devices to be tested)?

5.1.3.3 Can the sensors be placed so as to ensure equal exposure? This may require mounting the sensors on a rotating

fixture in steady-state irradiations or performing multiple irradiations with monitor foils to normalize the fluence between runs.

5.1.3.4 Do the DUT or the spectrum sensors perturb the neutron spectrum?

5.1.3.5 Are response functions available that account for self-shielding for all sensors using (n,γ) or non-threshold (n,f) reactions, unless the material is available in a dilute form of certified composition?

5.1.3.6 Can the fluence and spectrum seen in the DUT test later be directly scaled to that determined in the spectrum characterization experiment (by monitors placed with the tested device)?

5.1.3.7 Can the spectrum shape and intensity be characterized by integral parameters that permit simple intercomparison of device responses in different environments? Silicon is a semiconductor material whose displacement damage function is well established. This makes spectrum parameterization for damage predictions feasible for silicon.

5.1.3.8 What region of the spectrum contributes to the response of the DUT? In other words, is the spectrum well determined in all energy regions that affect device performance?

5.1.3.9 How is the counting system set up for the determination of the activities? For example, are there enough counters available to handle up to 25 reactions from a single exposure? (This may require as many as six counters.) Or can the available system only handle a few reactions before the activities have decayed below detectable limits?

5.1.4 Once the experimental opportunities and constraints have been addressed and the experiment designed to gather the most useful data, a spectrum adjustment methodology must be chosen.

## 5.2 Spectrum Adjustment Methodology:

5.2.1 After the basic measured responses, response functions, and trial or prior spectrum information have been assembled, apply a suitable spectrum adjustment procedure to reach a solution that satisfies the criteria of the chosen procedure. It must also meet other constraints such as, the fluence spectrum must be positive and defined for all energies. The solution is the energy-dependent spectrum function,  $\Phi(E)$ , which approximately satisfies the series of Fredholm equations of the first kind represented by Eq 1 as follows:

$$R_j = \int_0^{\infty} \sigma_j(E) \Phi(E) dE \quad 1 \leq j \leq n \quad (1)$$

where:

$R_j$  = measured response of sensor  $j$ ,  
 $\sigma_j(E)$  = neutron response function at energy  $E$  for sensor  $j$ ,

$\Phi(E)$  = incident neutron fluence versus energy, and  
 $n$  = number of sensors which yield  $n$  equations.

NOTE 3—Guides E720 and E844 provide general guidance on obtaining a suitable set of responses (activities) when foil monitors are used. Practice E261 and Test Method E262 provide more information on the data analysis that generally is part of an experiment with activation monitors. Specific instructions for some individual monitors can be found in Test Methods E263 (iron), E264 (nickel), E265 (sulfur-32), E266 (aluminum), E393 (barium-140 from fission foils), E523 (copper), E526

(titanium), E704 (uranium-238), E705 (neptunium-237), E1297 (niobium).

5.2.2 One important characteristic of the set of equations (Eq 1) is that with a finite number of sensors,  $n$ , which yield  $n$  equations, there is no unique solution. Exact solutions to equations (Eq 1) may be readily found, but are not generally considered useful. When the least squares adjustment method is used, equations (Eq 1) are supplemented by the constraint that the solution spectrum must be approximately equal to the prior spectrum. This additional constraint guarantees that the set of equation is overdetermined and that a unique least squares solution does exist. The tolerances of the approximations are dependent on the specified variances and covariances of the prior spectrum, the response functions, and the measured responses. When other adjustment methods are used it must be assumed that the range of physically reasonable solutions can be limited to an acceptable degree.

5.2.3 Neutron spectra generated from sensor response data may be obtained with several types of spectrum adjustment codes. One type is linear least squares minimization used by codes such as STAY'SL (2) or the logarithmic least squares minimization as used by LSL-M2 (3). When the spectrum adjustments are small, these methods yield almost identical results. Another type is the iterative method, an example of which is SAND II (4). If used properly and with sufficient, high-quality data, this method will usually yield nearly the same values as the least squares methods ( $\pm 10$  to 15 %) for the primary integral parameters discussed in E722.

NOTE 4—Another class of codes often referred to as Maximum Entropy (5) has also been used for this type of analysis.

5.2.4 Appendix X1 and Appendix X2 discuss in some detail the implementation and the advantages and disadvantages of the two approaches as represented by LSL-M2 and SAND-II.

## 5.3 Least-Squares Code Characteristics:

5.3.1 The least-squares codes, represented by STAY'SL (2) and LSL-M2 (3), use variance and covariance data for the measured responses, response functions, and prior spectrum. The STAY'SL code finds the unique maximum likelihood solution spectrum when the uncertainties are assumed to be distributed according to a normal distribution. The LSL-M2 code finds the unique maximum likelihood solution spectrum when the uncertainties are assumed to be distributed according to a multivariate lognormal distribution. The codes allow not only the prior spectrum but also the responses and the response functions to be adjusted in a manner constrained by their individual uncertainties and correlations in order to find the most likely solution. In principle this approach provides the best estimate of a spectrum and its uncertainties. The least-squares method is described more fully in Guide E944 and in Appendix X2.

5.3.2 The problem to the linear least squares spectrum adjustment problem is given in matrix form by:

$$\Phi' - \Phi_0 = C_{\Phi_0} S_{\Phi_0}^T (S_{\Phi_0} C_{\Phi_0} S_{\Phi_0}^T + S_{\Sigma_0} C_{\Sigma_0} S_{\Sigma_0}^T + C_{R_m})^{-1} (R_m - R_0) \quad (2)$$

where:

$\Phi'$  = a column vector of the adjusted groupwise fluences,

- $\Phi_0$  = a column vector of the prior spectrum fluences,  
 $C_{\Phi_0}$  = the covariance matrix of the prior spectrum,  
 $S_{\Phi_0}$  = the matrix of sensitivities of the calculated responses to the prior fluences,  
 $S_{\Sigma_0}$  = the matrix of sensitivities of the calculated responses to the response functions,  
 $C_{\Sigma_0}$  = the covariance matrix of the response functions,  
 $C_{R_m}$  = the covariance matrix of the measured responses,  
 $R_m$  = a column vector of the measured responses,  
 $R_0$  = a column vector of the responses calculated using the prior spectrum and response functions,

superscript  $T$  indicates the transpose of a matrix, and superscript  $-1$  indicates the inverse of a matrix.

5.3.3 Further details, including the sensitivity matrix entries, may be found in (2)). In the case of logarithmic least squares, the fluences and the responses are replaced by their natural logarithms, and the covariance matrices are replaced by the fractional covariance matrices, see (3)).

5.3.4 The input variance and covariance matrix quantities are not always well known and some may have to be estimated. The analyst must understand that his estimates of these quantities can affect the results.

5.3.5 No least-squares code in the form distributed by code libraries conveniently handles the effects of covers over the foils even though the use of covers is strongly recommended. See Section 7.2 of Guide E720 and X1.5.1 of this standard for more information.

5.3.6 The code automatically weights the data according to uncertainties. Therefore, data with large uncertainties can be used in the analysis, and will have the appropriately small influence on the results.

5.3.7 The solution spectrum shape must correspond fairly well to the prior spectrum (within 1 or 2 standard deviations) if the results are to be reliable (6). The prior spectrum determines the solution spectrum when its uncertainties are so small that the uncertainties of the prior calculated responses are small compared to those of the measured responses. Conversely, the prior spectrum does not strongly constrain the solution spectrum when the prior calculated responses are large compared to the measured responses. See Ref (3).

5.3.8 If a transport code calculation of the spectrum is used as the starting point for the analysis, then this methodology can be useful for adjusting spectra at a different location from that in which the foils were exposed. If the transport calculation includes a location where an experiment can be conducted and a similar one where such an experiment would be difficult or impossible (such as inside a test fixture or other structure), then this type of code can be used to adjust both spectra simultaneously. In accepting the results for the unmonitored location, it is important that the transport calculation be adjusted minimally.

5.3.9 The analyst must be careful the input variances and covariances, including those associated with the prior spectrum, are realistic. It is not sufficient to take statistical scoring errors from a Monte Carlo transport calculation and use these as a measure of the uncertainty in the trial spectrum. All uncertainties, and in particular, uncertainties in the reactor modeling, material densities, and response functions should be

represented in the input uncertainty. The value of the chi-squared ( $\chi^2$ ) parameter may be used as a good indication of the consistency of the input data (including the uncertainty data).

5.4 *Suitability of the Least-Squares Adjustment Codes*—The least-squares codes are particularly well suited to situations in which the environment is fairly well characterized physically so that a prior spectrum can be calculated. They work best when detailed transport calculated spectra are available for use as the prior spectra for the analysis. However, it is often difficult to obtain a mathematically defensible covariance matrix for these spectra. In such cases, the specified uncertainties in the prior spectrum should be large enough to ensure that the measured response uncertainties are larger than their calculated prior uncertainties. In principle, a sensitivity analysis based on the radiation transport code methodology could be used to provide the prior spectrum uncertainty and energy-dependent correlation, but this is not an easy analysis and is seldom attempted.

#### 5.5 *Iterative Code Characteristics:*

5.5.1 The “iterative” codes use a trial function supplied by the analyst and integrate it over the response functions of the sensors exposed in the unknown environment to predict a set of calculated responses for comparison with the measured values. The calculated responses are obtained from Eq 1. The code obtains the response functions from a library. See Guide E1018 for the recommendations in the selection of dosimetry-quality cross sections.

5.5.2 The code compares the measured and calculated responses for each effect and invokes an algorithm designed to alter the trial function so as to reduce the deviations between the measured and calculated responses. The process is repeated with code-altered spectra until the standard deviation drops below a specified value – at which time the coded declares that a solution has been obtained and prepares a table of the last spectrum. This should not be the end of the process unless the initial trial was very close to the final result. In each iteration, the SAND II-type code will alter the trial most rapidly where the foil set has the highest response. If the trial is incompatible with the measurements, the spectrum can become distorted in a very unphysical manner.

5.5.3 For example, if a trial function predicts an incorrect gold activity, it may alter the spectrum by orders of magnitude at the gold high-response resonance at 5 eV while leaving the trial spectrum alone in the immediate vicinity. The analyst must recognize that the trial must be changed in a manner suggested by the previous result. For example, if a peak develops at the gold resonance, this suggests that the trial spectrum values are too low in that whole energy region. A new trial drawn smoothly near the spectrum values where the sensor set has high response may improve the solution. This direct modification becomes an outer iteration on the spectrum adjustment process, as described in Refs (7, 8). The outer iteration methodology coupled with good activity data is usually so successful that the form of the initial trial does not overly influence the integral results.

5.5.4 For any of the iterative type codes to succeed at producing a spectrum that is both representative of the measured data and likely to be close to the true spectrum of

neutrons that caused the activation data, experience has shown that the following are important (1) the use of sensors with well-established response functions ( $\leq 8\%$  for spectrum-averaged cross sections), (2) a sensor set that has good response over all the important regions of the spectrum, and (3) sufficiently accurate measured responses (on the order of  $\pm 5\%$ ). No direct use is made of uncertainty data (variance and covariance information) that exists for each cross section, of uncertainty in the trial spectrum, or in the uncertainties in the measured responses. These uncertainties can vary greatly among sensors or environments. It follows that data with large uncertainties should not be used in the final stages of this methodology because it can cripple the final results.

NOTE 5—Response data that exhibits a strong disagreement with other data in the data ensemble can be very useful in the early stages of an analysis. For example, if the activity of a particular reaction is incompatible with the other foils in the spectrum adjustment process, it can indicate one of two important possibilities. First, if it is a reaction whose energy-dependent cross section is well known and has repeatedly demonstrated compatibility in the past, an experimental or transcription error is suggested. Second, if the activity measurement was accurately carried out, and this reaction has repeatedly demonstrated incompatibility in the same direction in other spectra determinations in different environments, an incorrect cross section or energy-specific counting calibration error is indicated (8). A number of specific cross section problems have been uncovered by analysis of incompatibility data. But in the construction of the neutron spectrum, these “bad” reactions should not be used with a method that does not incorporate uncertainty data.

### 5.6 Suitability of the Iterative Adjustment Codes:

5.6.1 Iterative codes usually do not have a capability to weight the responses according to uncertainties, do not provide error or uncertainty analysis, do not use variance or covariance information, and provide no direct quantification of the output uncertainties for any calculated quantities. However, it is possible to assign errors in the spectrum in appropriate energy regions using perturbation analysis. (Also computerized perturbation and random draw from response error may be utilized.) The analyst perturbs the trial spectrum upwards and downwards in each energy region and observes to what degree the code brings the two trials into agreement. This is, however, a laborious process and has to be interpreted carefully. In the resonance region where foil responses are spiked, the code will only yield agreement at resonances where there exists high response. The analyst must not only interpolate the spectrum values between high response regions but also the spectrum uncertainties. This step can be rationalized with physical arguments based on the energy-dependence of cross sections but it is difficult to justify mathematically. This situation further supports the arguments for maximizing response coverage. In addition, it is usually the uncertainties of integral parameters that are of primary importance, not the uncertainty of  $\Phi(E)$  at individual energy values

5.6.2 Covers are used over many of the foils to restrict the response ranges, as is explained in Guide E720. The SAND II code handles the attenuations in the covers in a simple manner by assuming exponential attenuation through the cover material. There is considerable evidence that for some spectra the calculated exponential attenuation is not accurate because of scattering.

## 6. Discussion and Comparison of Methodology Characteristics

6.1 The least-squares codes are superior because it should be possible to directly incorporate all that is known about the test environment and about the response functions to arrive at the most likely solution in a least-squares sense. The codes provide mathematically defensible output with uncertainties when covariance data is available for all the input quantities. The iterative codes do not propagate uncertainties nor make use of any variance or covariance information which may exist.

6.2 Considerable experience with both approaches has demonstrated that they yield approximately the same integral parameter values when applied to the methodology in E722, provided that adequate and accurate primary experimental information is available. This means the analyst must have access to a set of carefully measured responses, usually activation data. The associated set of responses functions, usually activation cross sections, must cover a broad range of energies. And, the response functions for the measured data must be well established over these energy ranges.

6.3 Transient radiation effects testing of electronics (TREE testing) is carried out in a wide variety of different environments that are often customized with complicated filters and shields. For these cases, detailed transport calculations can be time-consuming and expensive. The user may not be aware of the total assemblage of material structure that affects the radiation environment.

6.4 The iterative type code performs at its best with accurate response data and well-known response functions because the range of acceptable solutions is then severely restricted, and the acceptance criterion of measured-to-calculated activity values can be set to a low value. Also, incompatible responses, perhaps caused by experimental errors, stand out clearly in the results. The least-squares type code seems much more forgiving because wide variances are assigned to less well-known cross sections and activities, so marginal data can be more easily tolerated. For both methods, a very good trial function or prior spectrum is required when limited or imprecise measured responses are available. In these cases, the solution cannot be allowed to deviate very much from the trial because less use should be made of the measured data.

6.5 SAND II should not be used to generate trial functions for LSL-M2, because the SAND II solution spectrum is correlated to the activities, but the LSL method assumes there is no such correlation.

6.6 Neither methodology can be used indiscriminately and without careful monitoring by a knowledgeable analyst. The analyst must not only apply physical reasoning but must examine the data to determine if it is of adequate quality. At the very least the analyst must evaluate what is seen in a plot of the solution spectrum. Available versions of the SAND II code provides less subsidiary information than least-squares codes can supply, particularly with regards to uncertainties. More detailed discussions of the LSL-M2 and SAND II methodologies are provided in the appendices.

## 7. Precision and Bias

7.1 Precision and bias statements are included in each of the appendixes.

## 8. Keywords

8.1 neutron sensors; neutron spectra; radiation-hardness testing; spectrum adjustment

## APPENDIXES

### (Nonmandatory Information)

#### X1. APPLICATION OF THE LSL-M2 CODE

##### X1.1 The Least-Squares Method, LSL-M2

X1.1.1 This appendix provides guidance for the application of the LSL-M2 adjustment code to hardness testing of electronic devices. The code is described in Refs (9) and (10). However, it is designed for commercial power reactor pressure vessel surveillance applications and the documentation was developed accordingly. This appendix provides guidance for those circumstances where the documentation is inadequate or inappropriate for hardness-testing applications.

##### X1.2 Introduction

X1.2.1 As Eq 1 implies, three basic data sets are required in the determination of the neutron energy-fluence spectrum: (1) a set of measured responses (see Guide E720 for guidance on foil selection), (2) energy-response functions, and (3) an approximation to the solution.

X1.2.2 Several codes have been developed which implement a least-squares approach to the determination of the neutron spectrum from sensor data. The least-squares codes require a minimum of three additional data sets in the form of uncertainty estimates for all the above data, complete with the correlations between all the data. These additional data are used to establish uncertainty estimates on the output data. See Section 3.2 of Guide E944 for more information.

X1.2.3 The LSL-M2 code (10) is one example of a least square code package which is distributed with a suitable set of auxiliary data (cross sections and covariance files) to permit its application for the adjustment of reactor pressure vessel neutron spectra. As part of the REAL exercises (11, 12, 13) the International Atomic Energy Agency (IAEA) compiled and distributed a Neutron Metrology File NMF-90 (14) which includes versions of the MIEKE (15) and STAY'SL (2) least square adjustment codes along with compatible cross sections and sample input decks. These three codes are examples of least square adjustment codes which are available to the general community and include interfaces with suitable cross section libraries.

X1.2.4 An adequate prior or theoretical prediction of the fluence spectrum (with its covariance matrix) is often the most difficult information set to obtain. If a transport calculation is available, it may be a generic type of run such as a leakage spectrum from the reactor or a criticality calculation that provides a typical spectrum for some location.

X1.2.5 An error estimate of the group-wise fluences, with correlations, is essential to LSL-M2, but is not always readily

available to the analyst. The error analysis distributed with the code may be applied, with caution, to pool-type reactors if nothing else is available, but it is not applicable to fast-burst reactors or <sup>252</sup>Cf sources and should not be used. However, the LSL-M2 code can be applied to most reactors used for testing of electronic devices whether an error estimate of the spectrum is available or not. The practical aspects of this will be described in X1.4.

##### X1.3 Constraints on the Use of the Code

X1.3.1 The LSL-M2 code is distinguished by its use of lognormal distributions for all the parameters of interest. This imposes the physically realistic constraint that all quantities are positive and real. The formulation of the equations described in Section 3 of Guide E944 were all converted to logarithmic counterparts by the writers of LSL-M2. As stated in the manual, care should be taken to input covariances as fractional covariances: the expected values of  $\frac{\delta x_i \delta x_j}{x_i x_j}$ . In the same fashion, the output uncertainties are actually logarithmic ratios of the standard deviation to the expected value. The primary output of LSL-M2 is not the adjusted spectrum, but rather the damage-related integral parameters with their errors. This feature is ideally suited to the calculation of silicon damage as defined in Practice E722.

NOTE X1.1—There is little difference between these logarithmic ratios and the more normal values if the percentages quoted are less than 10 %. But, as the ratio of (observed/actual) increases, the LSL ratio diverges from the non-logarithmic ratio.

X1.3.1.1 Dosimetry cross-section sets and associated covariance matrices are available with the LSL-M2 code package. The cross sections distributed by RSICC with this code have been derived from ENDF/B-V evaluations (10). The newer IRDF-2002 distribution (16) is recommended as a replacement for the code's build-in data sets. In most cases, it contains newer evaluations for the actual cross sections and much more refined evaluations of the associated covariance data for each reaction.

X1.3.1.2 The response data is obtained through the application of Guides E720 and E844, Practice E261 and Test Methods E262, E263, E264, E265, E393, E704, and E705. The uncertainty estimate for each response should not be simply an estimate of the counting statistics, but rather should include all contributors of uncertainty to the measured value, such as uncertainties in counting efficiency, branching ratio, foil composition, mass, experimental positioning, etc. Correlations between reactions may be important, particularly when the same radioactive product is measured on the same detector.